

ENGINEERING CHANGE NOTICE

Page 1 of 2

1. ECN 635447

Proj.
ECN

2. ECN Category (mark one) Supplemental <input type="checkbox"/> Direct Revision <input checked="" type="checkbox"/> Change ECN <input type="checkbox"/> Temporary <input type="checkbox"/> Standby <input type="checkbox"/> Supersedeure <input type="checkbox"/> Cancel/Void <input type="checkbox"/>	3. Originator's Name, Organization, MSIN, and Telephone No. Leela M. Sasaki, Data Assessment and Interpretation, R2-12, 373-1027	4. USQ Required? <input type="checkbox"/> Yes <input checked="" type="checkbox"/> No	5. Date 03/10/97	
	6. Project Title/No./Work Order No. Tank 241-T-107	7. Bldg./Sys./Fac. No. 241-T-107	8. Approval Designator N/A	
	9. Document Numbers Changed by this ECN (includes sheet no. and rev.) WHC-SD-WM-ER-382, Rev. 0-A	10. Related ECN No(s). ECN-614165	11. Related PO No. N/A	

12a. Modification Work <input type="checkbox"/> Yes (fill out Blk. 12b) <input checked="" type="checkbox"/> No (NA Blks. 12b, 12c, 12d)	12b. Work Package No. N/A	12c. Modification Work Complete N/A Design Authority/Cog. Engineer Signature & Date	12d. Restored to Original Condition (Temp. or Standby ECN only) N/A Design Authority/Cog. Engineer Signature & Date
---	------------------------------	---	---

13a. Description of Change

13b. Design Baseline Document? Yes No

This ECN was generated in order to revise the document to the new format per Department of Energy performance agreements.

14a. Justification (mark one)

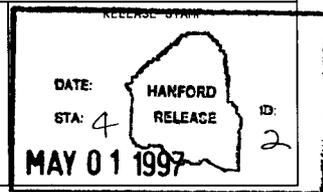
Criteria Change <input type="checkbox"/>	Design Improvement <input type="checkbox"/>	Environmental <input type="checkbox"/>	Facility Deactivation <input type="checkbox"/>
As-Found <input checked="" type="checkbox"/>	Facilitate Const <input type="checkbox"/>	Const. Error/Omission <input type="checkbox"/>	Design Error/Omission <input type="checkbox"/>

14b. Justification Details

This document was revised per Department of Energy performance agreements and direction from the Washington State Department of Ecology to revise 23 tank characterization reports (letter dated 7/6/95).

15. Distribution (include name, MSIN, and no. of copies)

See attached distribution.



ENGINEERING CHANGE NOTICE

Page 2 of 2

1. ECN (use no. from pg. 1)

ECN-635447

16. Design Verification Required

Yes
 No

17. Cost Impact

ENGINEERING

Additional \$
Savings \$

CONSTRUCTION

Additional \$
Savings \$

18. Schedule Impact (days)

Improvement
Delay

19. Change Impact Review: Indicate the related documents (other than the engineering documents identified on Side 1) that will be affected by the change described in Block 13. Enter the affected document number in Block 20.

SDD/DD	<input type="checkbox"/>	Seismic/Stress Analysis	<input type="checkbox"/>	Tank Calibration Manual	<input type="checkbox"/>
Functional Design Criteria	<input type="checkbox"/>	Stress/Design Report	<input type="checkbox"/>	Health Physics Procedure	<input type="checkbox"/>
Operating Specification	<input type="checkbox"/>	Interface Control Drawing	<input type="checkbox"/>	Spares Multiple Unit Listing	<input type="checkbox"/>
Criticality Specification	<input type="checkbox"/>	Calibration Procedure	<input type="checkbox"/>	Test Procedures/Specification	<input type="checkbox"/>
Conceptual Design Report	<input type="checkbox"/>	Installation Procedure	<input type="checkbox"/>	Component Index	<input type="checkbox"/>
Equipment Spec.	<input type="checkbox"/>	Maintenance Procedure	<input type="checkbox"/>	ASME Coded Item	<input type="checkbox"/>
Const. Spec.	<input type="checkbox"/>	Engineering Procedure	<input type="checkbox"/>	Human Factor Consideration	<input type="checkbox"/>
Procurement Spec.	<input type="checkbox"/>	Operating Instruction	<input type="checkbox"/>	Computer Software	<input type="checkbox"/>
Vendor Information	<input type="checkbox"/>	Operating Procedure	<input type="checkbox"/>	Electric Circuit Schedule	<input type="checkbox"/>
OM Manual	<input type="checkbox"/>	Operational Safety Requirement	<input type="checkbox"/>	ICRS Procedure	<input type="checkbox"/>
FSAR/SAR	<input type="checkbox"/>	IEFD Drawing	<input type="checkbox"/>	Process Control Manual/Plan	<input type="checkbox"/>
Safety Equipment List	<input type="checkbox"/>	Cell Arrangement Drawing	<input type="checkbox"/>	Process Flow Chart	<input type="checkbox"/>
Radiation Work Permit	<input type="checkbox"/>	Essential Material Specification	<input type="checkbox"/>	Purchase Requisition	<input type="checkbox"/>
Environmental Impact Statement	<input type="checkbox"/>	Fac. Proc. Samp. Schedule	<input type="checkbox"/>	Tickler File	<input type="checkbox"/>
Environmental Report	<input type="checkbox"/>	Inspection Plan	<input type="checkbox"/>		<input type="checkbox"/>
Environmental Permit	<input type="checkbox"/>	Inventory Adjustment Request	<input type="checkbox"/>		<input type="checkbox"/>

20. Other Affected Documents: (NOTE: Documents listed below will not be revised by this ECN.) Signatures below indicate that the signing organization has been notified of other affected documents listed below.

Document Number/Revision	Document Number/Revision	Document Number/Revision
N/A		

21. Approvals

	Signature	Date		Signature	Date
Design Authority			Design Agent		
Cog. Eng. L.M. Sasaki	<i>L.M. Sasaki</i>	<u>4/30/97</u>	PE		
Cog. Mgr. K.M. Hall	<i>Kathleen M. Hall</i>	<u>4/30/97</u>	QA		
QA			Safety		
Safety			Design		
Environ.			Environ.		
Other R.J. Cash	<i>R.J. Cash</i>	<u>4/30/97</u>	Other		
N.W. Kirch	<i>N.W. Kirch</i>	<u>4/30/97</u>			
			DEPARTMENT OF ENERGY		
			Signature or a Control Number that tracks the Approval Signature		
			ADDITIONAL		

Tank Characterization Report for Single-Shell Tank 241-T-107

Leela M. Sasaki

Lockheed Martin Hanford Corp., Richland, WA 99352
U.S. Department of Energy Contract DE-AC06-87RL10930

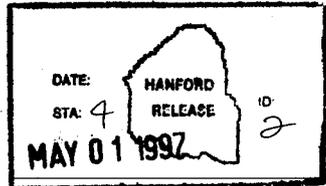
EDT/ECN: ECN-635447 UC: 2070
Org Code: 74620 Charge Code: N4G4C
B&R Code: EW 3120074 Total Pages: 262

Key Words: Waste Characterization, Single-Shell Tank, SST, Tank 241-T-107, Tank T-107, T-107, T Farm, Tank Characterization Report, TCR, Waste Inventory, TPA Milestone M-44

Abstract: This document summarizes the information on the historical uses, present status, and the sampling and analysis results of waste stored in Tank 241-T-107. This report supports the requirements of the Tri-Party Agreement Milestone M-44-05.

TRADEMARK DISCLAIMER. Reference herein to any specific commercial product, process, or service by trade name, trademark, manufacturer, or otherwise, does not necessarily constitute or imply its endorsement, recommendation, or favoring by the United States Government or any agency thereof or its contractors or subcontractors.

Printed in the United States of America. To obtain copies of this document, contact: WHC/BCS Document Control Services, P.O. Box 1970, Mailstop H6-08, Richland WA 99352, Phone (509) 372-2420; Fax (509) 376-4989.



Leela M. Sasaki 7/1/97
Release Approval Date

Approved for Public Release

Tank Characterization Report for Single-Shell Tank 241-T-107

L. M. Sasaki
J. A. Lechelt
Lockheed Martin Hanford Corporation

N. L. Hulse
Los Alamos Technical Associates

S. R. Wilmarth
Numatec Hanford Corporation

Date Published
April 1997

Prepared for the U.S. Department of Energy
Assistant Secretary for Environmental Management

Project Hanford Management Contractor for the
U.S. Department of Energy under Contract DE-AC06-96RL13200

Approved for public release; distribution is unlimited.

CONTENTS

1.0 INTRODUCTION 1-1

 1.1 SCOPE 1-1

 1.2 TANK BACKGROUND 1-3

2.0 RESPONSE TO TECHNICAL ISSUES 2-1

 2.1 SAFETY SCREENING 2-1

 2.1.1 Exothermic Conditions (Energetics) 2-2

 2.1.2 Flammable Gas 2-2

 2.1.3 Criticality 2-2

 2.2 FERROCYANIDE ISSUES 2-3

 2.3 VAPOR SCREENING 2-3

 2.3.1 Flammable Gas 2-3

 2.3.2 Toxicity 2-3

 2.4 OTHER TECHNICAL ISSUES 2-4

 2.5 SUMMARY 2-5

3.0 BEST-BASIS INVENTORY ESTIMATE 3-1

4.0 RECOMMENDATIONS 4-1

5.0 REFERENCES 5-1

APPENDIXES

APPENDIX A: HISTORICAL TANK INFORMATION A-1

A1.0 CURRENT TANK STATUS A-3

A2.0 TANK DESIGN AND BACKGROUND A-4

A3.0 PROCESS KNOWLEDGE A-9

 A3.1 WASTE TRANSFER HISTORY A-9

 A3.2 HISTORICAL ESTIMATION OF TANK CONTENTS A-11

A4.0 SURVEILLANCE DATA A-14

 A4.1 SURFACE-LEVEL READINGS A-15

 A4.2 INTERNAL TANK TEMPERATURES A-15

 A4.3 TANK 241-T-107 PHOTOGRAPHS A-18

A5.0 APPENDIX A REFERENCES A-19

CONTENTS (Continued)

APPENDIX B: SAMPLING OF TANK 241-T-107	B-1
B1.0 TANK SAMPLING OVERVIEW	B-3
B2.0 SAMPLING EVENTS	B-5
B2.1 NOVEMBER 1992 - MARCH 1993 CORE SAMPLING EVENT	B-5
B2.1.1 Description of Sampling Event	B-5
B2.1.2 Sample Handling	B-6
B2.1.3 Sample Analysis	B-9
B2.1.4 Analytical Results	B-14
B2.2 OCTOBER 1992 VAPOR SAMPLES	B-128
B2.2.1 Description of Sampling Event	B-128
B2.2.2 Sample Handling	B-128
B2.2.3 Sample Analysis	B-128
B2.2.4 Analytical Results	B-129
B2.3 JANUARY 1995 VAPOR SAMPLES	B-129
B2.3.1 Description of Sampling Event	B-129
B2.3.2 Sample Handling	B-130
B2.3.3 Sample Analysis	B-130
B2.3.4 Analytical Results	B-130
B2.4 HISTORICAL SAMPLE RESULTS	B-131
B3.0 ASSESSMENT OF CHARACTERIZATION RESULTS	B-136
B3.1 FIELD OBSERVATIONS	B-136
B3.2 QUALITY CONTROL ASSESSMENT	B-137
B3.2.1 Quality Control Assessment for the 1992 and 1993 Core Sampling Event	B-137
B3.2.2 Quality Control Assessment for the 1995 Vapor Sampling Event	B-138
B3.3 DATA CONSISTENCY CHECKS	B-139
B3.3.1 Comparison of Results from Different Analytical Methods	B-139
B3.3.2 Mass and Charge Balances	B-142
B3.3.3 Review of the Analyte Profile	B-146
B3.3.4 Homogenization Test Description	B-148
B3.4 MEAN CONCENTRATIONS AND CONFIDENCE INTERVALS	B-149
B3.4.1 Mean Concentration Estimates	B-149
B3.4.2 Segment-Level Means	B-153
B3.4.3 Drainable Liquid Means	B-154
B3.4.4 ANOVA Models for Core Composite and Drainable Liquid Data	B-156
B3.4.5 ANOVA Model for Core Segment Data	B-157

CONTENTS (Continued)

B4.0 APPENDIX B REFERENCES B-160

APPENDIX C: STATISTICAL ANALYSIS FOR ISSUE RESOLUTION C-1

C1.0 APPENDIX C REFERENCES C-3

APPENDIX D: EVALUATION TO ESTABLISH BEST-BASIS INVENTORY
FOR SINGLE-SHELL TANK 241-T-107 D-1

D1.0 IDENTIFY/COMPILE INVENTORY SOURCES D-3

D2.0 COMPARE COMPONENT INVENTORY VALUES D-3

D3.0 REVIEW AND EVALUATION OF COMPONENT INVENTORIES D-5

 D3.1 CONTRIBUTING WASTE TYPES D-6

 D3.2 EVALUATION OF TECHNICAL INFORMATION D-6

 D3.3 ASSUMPTIONS FOR RECONCILING WASTE INVENTORIES D-8

 D3.4 CONCENTRATION FACTOR AND PARTITIONING FACTORS
 FOR TANK 241-T-107 D-9

 D3.5 CALCULATION OF ASSESSMENT-BASED INVENTORIES IN
 TANK 241-T-107 D-11

 D3.6 COMPARISON OF SELECTED INVENTORY ESTIMATES D-12

 D3.7 CONCLUSIONS D-14

D4.0 DEFINE THE BEST-BASIS AND ESTABLISH COMPONENT
 INVENTORIES D-15

D5.0 APPENDIX D REFERENCES D-17

APPENDIX E: BIBLIOGRAPHY FOR TANK 241-T-107 E-1

LIST OF FIGURES

A2-1 Riser Configuration for Tank 241-T-107 A-6

A2-2 Tank 241-T-107 Cross Section and Schematic A-7

A3-1 Tank 241-T-107 Tank Layer Model A-12

A4-1 Tank 241-T-107 Level History A-16

A4-2 Tank 241-T-107 High Temperature Plot A-17

LIST OF TABLES

1-1 Summary of Recent Sampling 1-2

1-2 Description of Tank 241-T-107 1-4

2-1 Tank 241-T-107 Projected Heat Load 2-4

2-2 Summary of Safety Screening, Vapor Screening, and Ferrocyanide
Evaluation Results 2-5

3-1 Best-Basis Inventory Estimates for Nonradioactive Components
in Tank 241-T-107 3-2

3-2 Best-Basis Inventory Estimates for Radioactive Components in Tank 241-T-107 . . 3-3

4-1 Acceptance of Tank 241-T-107 Sampling and Analysis 4-1

4-2 Acceptance of Evaluation of Characterization Data and Information
for Tank 241-T-107 4-2

A1-1 Tank Contents Status Summary A-4

A2-1 Tank 241-T-107 Risers A-8

A3-1 Tank 241-T-107 Major Transfers A-10

A3-2 Tank 241-T-107 Historical Tank Inventory Estimate A-13

LIST OF TABLES (Continued)

B1-1	Integrated Data Quality Objective Requirements for Tank 241-T-107	B-4
B2-1	Tank 241-T-107 Subsampling Scheme and Sample Description	B-7
B2-2	Solid Samples Preparation Summary	B-10
B2-3	Analytical Methods For Organic and Physical Analyses	B-12
B2-4	Analytical Methods for Chemical and Radionuclide Analyses	B-13
B2-5	Analytical Presentation Tables	B-14
B2-6	Physical Properties Summary (Core 50, Segment 2)	B-19
B2-7	Particle Size Data for Tank 241-T-107	B-22
B2-8	Tank 241-T-107 Analytical Results: Cesium (AA)	B-24
B2-9	Tank 241-T-107 Analytical Results: Mercury (CVAA)	B-24
B2-10	Tank 241-T-107 Analytical Results: Aluminum (ICP)	B-25
B2-11	Tank 241-T-107 Analytical Results: Antimony (ICP)	B-27
B2-12	Tank 241-T-107 Analytical Results: Arsenic (ICP)	B-29
B2-13	Tank 241-T-107 Analytical Results: Beryllium (ICP)	B-31
B2-14	Tank 241-T-107 Analytical Results: Bismuth (ICP)	B-33
B2-15	Tank 241-T-107 Analytical Results: Boron (ICP)	B-35
B2-16	Tank 241-T-107 Analytical Results: Cadmium (ICP)	B-37
B2-17	Tank 241-T-107 Analytical Results: Calcium (ICP)	B-39
B2-18	Tank 241-T-107 Analytical Results: Cerium (ICP)	B-41
B2-19	Tank 241-T-107 Analytical Results: Chromium (ICP)	B-43
B2-20	Tank 241-T-107 Analytical Results: Iron (ICP)	B-45

LIST OF TABLES (Continued)

B2-21	Tank 241-T-107 Analytical Results: Lanthanum (ICP)	B-47
B2-22	Tank 241-T-107 Analytical Results: Lead (ICP)	B-49
B2-23	Tank 241-T-107 Analytical Results: Lithium (ICP)	B-51
B2-24	Tank 241-T-107 Analytical Results: Magnesium (ICP)	B-53
B2-25	Tank 241-T-107 Analytical Results: Manganese (ICP)	B-55
B2-26	Tank 241-T-107 Analytical Results: Molybdenum (ICP)	B-57
B2-27	Tank 241-T-107 Analytical Results: Neodymium (ICP)	B-59
B2-28	Tank 241-T-107 Analytical Results: Nickel (ICP)	B-61
B2-29	Tank 241-T-107 Analytical Results: Phosphorus (ICP)	B-63
B2-30	Tank 241-T-107 Analytical Results: Potassium (ICP)	B-65
B2-31	Tank 241-T-107 Analytical Results: Samarium (ICP)	B-66
B2-32	Tank 241-T-107 Analytical Results: Selenium (ICP)	B-68
B2-33	Tank 241-T-107 Analytical Results: Silicon (ICP)	B-70
B2-34	Tank 241-T-107 Analytical Results: Silver (ICP)	B-72
B2-35	Tank 241-T-107 Analytical Results: Sodium (ICP)	B-74
B2-36	Tank 241-T-107 Analytical Results: Strontium (ICP)	B-76
B2-37	Tank 241-T-107 Analytical Results: Sulfur (ICP)	B-78
B2-38	Tank 241-T-107 Analytical Results: Thallium (ICP)	B-80
B2-39	Tank 241-T-107 Analytical Results: Titanium (ICP)	B-82
B2-40	Tank 241-T-107 Analytical Results: Zirconium (ICP)	B-84
B2-41	Tank 241-T-107 Analytical Results: Hexavalent Chromium (Spectrophotometry)	B-85

LIST OF TABLES (Continued)

B2-42A	Tank 241-T-107 Analytical Results: Total Uranium (LF)	B-86
B2-42B	Tank 241-T-107 Analytical Results: Uranium-238	B-86
B2-43	Tank 241-T-107 Analytical Results: Chloride (IC)	B-87
B2-44	Tank 241-T-107 Analytical Results: Fluoride (IC)	B-87
B2-45	Tank 241-T-107 Analytical Results: Nitrate (IC)	B-88
B2-46	Tank 241-T-107 Analytical Results: Nitrite (IC)	B-88
B2-47	Tank 241-T-107 Analytical Results: Phosphate (IC)	B-89
B2-48	Tank 241-T-107 Analytical Results: Sulfate (IC)	B-89
B2-49	Tank 241-T-107 Analytical Results: Ammonia (Distillation)	B-90
B2-50	Tank 241-T-107 Analytical Results: Nitrite (Spectrophotometry)	B-90
B2-51	Tank 241-T-107 Analytical Results: Cyanide (Spectrometry)	B-91
B2-52A	Tank 241-T-107 Analytical Results: Semivolatile Organic Analysis Detected Results	B-92
B2-52B	Tank 241-T-107 Analytical Results: Semivolatile Organic Analysis Nondetected Results	B-94
B2-53	Tank 241-T-107 Analytical Results: Total Carbon (Persulfate Oxidation)	B-99
B2-54A	Tank 241-T-107 Analytical Results: Total Organic Carbon (Persulfate Oxidation)	B-100
B2-54B	Tank 241-T-107 Analytical Results: Total Organic Carbon (Furnace Oxidation)	B-100
B2-55A	Tank 241-T-107 Analytical Results: Total Inorganic Carbon (Persulfate Oxidation)	B-101
B2-55B	Tank 241-T-107 Analytical Results: Total Inorganic Carbon (Furnace Oxidation)	B-101

LIST OF TABLES (Continued)

B2-56A	Tank 241-T-107 Analytical Results: Total Alpha (Alpha Proportional Counting)	B-102
B2-56B	Tank 241-T-107 Analytical Results: Total Alpha Pu (Alpha Proportional Counting)	B-102
B2-57	Tank 241-T-107 Analytical Results: Total Beta (Beta Proportional counting)	B-103
B2-58	Tank 241-T-107 Analytical Results: Strontium-90 (Beta Proportional Counting)	B-103
B2-59	Tank 241-T-107 Analytical Results: Americium-241 (Alpha Spectrometry)	B-104
B2-60	Tank 241-T-107 Analytical Results: Plutonium-238 (Alpha Spectrometry)	B-105
B2-61	Tank 241-T-107 Analytical Results: Plutonium-239/40 (Alpha Spectrometry)	B-105
B2-62	Tank 241-T-107 Analytical Results: Plutonium-238 to Pu Ratio (Mass Spectrometry)	B-106
B2-63	Tank 241-T-107 Analytical Results: Plutonium-239 to Pu Ratio (Mass Spectrometry)	B-106
B2-64	Tank 241-T-107 Analytical Results: Plutonium-240 to Pu Ratio (Mass Spectrometry)	B-106
B2-65	Tank 241-T-107 Analytical Results: Plutonium-241 to Pu Ratio (Mass Spectrometry)	B-106
B2-66	Tank 241-T-107 Analytical Results: Plutonium-242 to Pu Ratio (Mass Spectrometry)	B-107
B2-67	Tank 241-T-107 Analytical Results: Uranium-234 to U Ratio (Mass Spectrometry)	B-107
B2-68	Tank 241-T-107 Analytical Results: Uranium-235 to U Ratio (Mass Spectrometry)	B-107

LIST OF TABLES (Continued)

B2-69	Tank 241-T-107 Analytical Results: Uranium-236 to U Ratio (Mass Spectrometry)	B-108
B2-70	Tank 241-T-107 Analytical Results: Uranium-238 to U Ratio (Mass Spectrometry)	B-108
B2-71	Tank 241-T-107 Analytical Results: Americium-241 (GEA)	B-109
B2-72	Tank 241-T-107 Analytical Results: Cerium/Praeseodymium-144 (GEA)	B-110
B2-73	Tank 241-T-107 Analytical Results: Cesium-134 (GEA)	B-111
B2-74	Tank 241-T-107 Analytical Results: Cesium-137 (GEA)	B-112
B2-75	Tank 241-T-107 Analytical Results: Cobalt-60 (GEA)	B-113
B2-76	Tank 241-T-107 Analytical Results: Europium-154 (GEA)	B-114
B2-77	Tank 241-T-107 Analytical Results: Europium-155 (GEA)	B-115
B2-78	Tank 241-T-107 Analytical Results: Potassium-40 (GEA)	B-116
B2-79	Tank 241-T-107 Analytical Results: Ruthenium-103 (GEA)	B-117
B2-80	Tank 241-T-107 Analytical Results: Ruthenium/Rhodium-106 (GEA)	B-118
B2-81	Tank 241-T-107 Analytical Results: Thorium-228 (GEA)	B-119
B2-82	Tank 241-T-107 Analytical Results: Iodine-129 (Low Energy GEA)	B-120
B2-83	Tank 241-T-107 Analytical Results: Carbon-14 (Liquid Scintillation)	B-120
B2-84	Tank 241-T-107 Analytical Results: Tritium (Liquid Scintillation)	B-121
B2-85	Tank 241-T-107 Analytical Results: Technetium-99 (Liquid Scintillation)	B-121
B2-86	Tank 241-T-107 Analytical Results: Bulk Density	B-122
B2-87	Tank 241-T-107 Analytical Results: Density	B-123
B2-88	Tank 241-T-107 Analytical Results: Specific Gravity	B-123

LIST OF TABLES (Continued)

B2-89	Tank 241-T-107 Analytical Results: pH Measurement	B-124
B2-90	Tank 241-T-107 Analytical Results: Weight Percent Solids (Gravimetry) . . .	B-125
B2-91	Tank 241-T-107 Analytical Results: Weight Percent Residual Solids	B-125
B2-92	Tank 241-T-107 Analytical Results: Residual Solids	B-126
B2-93	Tank 241-T-107 Analytical Results: Total Dissolved Solids	B-126
B2-94	Tank 241-T-107 Analytical Results: Shear Strength	B-126
B2-95	Tank 241-T-107 Analytical Results: Rheological Parameters	B-127
B2-96	Tank 241-T-107 Analytical Results: Percent Water (TGA)	B-127
B2-97	Summary Results of Vapor Samples Collected from the Headspace of Tank 241-T-107 on October 22, 1992	B-129
B2-98	Summary Results of Vapor Samples Collected from the Headspace of Tank 241-T-107 on January 18, 1995	B-130
B2-99	September 1965 Supernatant Sample	B-132
B2-100	September 1975 Supernatant Sample	B-133
B2-101	March 1985 Supernatant Sample	B-134
B2-102	August 1989 Supernatant Sample	B-135
B3-1	Total Beta Comparison	B-141
B3-2	Comparison of Percent Water Results From Thermogravimetric and Gravimetric Analyses	B-141
B3-3	Cation Mass and Charge Data	B-144
B3-4	Anion Mass and Charge Data	B-145
B3-5	Mass Balance Totals	B-145
B3-6	Core Composite Statistics	B-150

LIST OF TABLES (Continued)

B3-7 Segment Data Concentration Estimate Statistics B-153

B3-8 Drainable Liquid Composite Data Concentration Estimate Statistics B-154

D2-1 Sampling-Based and Hanford Defined Waste-Based Inventory Estimates
for Nonradioactive Components in Tank 241-T-107 D-4

D2-2 Sampling and Hanford Defined Waste Predicted Inventory Estimates
for Radioactive Components in Tank 241-T-107 D-5

D3-1 Expected Solids for Tank 241-T-107 D-6

D3-2 Technical Flowsheet and Hanford Defined Waste Streams D-7

D3-3 Comparison of Selected Inventory Estimates for Tank 241-T-107 Waste D-12

D4-1 Best Basis Inventory Estimates for Nonradioactive Components
in Tank 241-T-107 D-15

D4-2 Best Basis Inventory Estimates for Radioactive Components in
Tank 241-T-107 D-16

LIST OF TERMS

1C	first-cycle decontamination waste
1C1	first-cycle decontamination waste generated from 1944 to 1949
AA	atomic absorption
ANOVA	analysis of variance
APC	alpha proportional counting
BPC	beta proportional counting
Btu/hr	British thermal units per hour
CF	concentration factor
Ci	curie
Ci/g	curies per gram
Ci/L	curies per liter
cm	centimeter
cm ²	square centimeter
cP	centipoise
CVAA	cold vapor atomic absorption
CW	cladding waste
DOE	U.S. Department of Energy
DQO	data quality objective
DSC	differential scanning calorimetry
dynes/cm ²	dynes per square centimeter
Ecology	Washington State Department of Ecology
FIC	Food Instrument Corporation
ft	feet
g	gram
g C/L	grams of carbon per liter
g/gal	grams per gallon
GC	gas chromatography
GEA	gamma energy analysis
g/L	grams per liter
g/mL	grams per milliliter
g-mol	gram-mole
HDW	Hanford Defined Waste
IC	ion chromatography
ICP	inductively coupled plasma spectroscopy
in.	inch
IX	ion exchange waste
kg	kilogram
kgal	kilogallon
kL	kiloliter
LEL	lower explosive limit
LF	laser fluorimetry
LFL	lower flammability limit

LIST OF TERMS (Continued)

LL	lower limit
m	meter
M	moles per liter
mg	milligram
mg/L	milligrams per liter
mg/m ³	milligrams per cubic meter
mL	milliliters
mL/in.	milliliters per inch
mm	millimeter
mrad/hr	millirad per hour
MS	mass spectrometry
n/a	not applicable
NA	not analyzed
N/D	not determined
n/r	not reported
NR	not required
Pa	Pascals
Pa/sec	Pascals per second
PF	partitioning factor
PHMC	Project Hanford Management Contractor
ppmv	parts per million by volume
PUREX	plutonium-uranium extraction plant
QC	quality control
rad/hr	rads per hour
REML	restricted maximum likelihood estimation
RPD	relative percent difference
RSD	relative standard deviation
SMM	supernatant mixing model
SORWT	Sort of Radioactive Waste Types (model)
SVOA	semivolatle organic analysis
TBP	tributyl phosphate
TC	total carbon
TCR	tank characterization report
TGA	thermogravimetric analysis
TIC	total inorganic carbon
TLM	tank layer model
TOC	total organic carbon
TWRS	Tank Waste Remediation System
UL	upper limit
UR	uranium recovery waste
vol%	volume percent
W	watt

LIST OF TERMS (Continued)

WSTRS	Waste Status and Transaction Record Summary
wt%	weight percent
°C	degrees Celsius
°F	degrees Fahrenheit
%	percent
μCi/g	microcuries per gram
μCi/gal	microcuries per gallon
μCi/mL	microcuries per milliliter
μm	micrometer
μCi/L	microcuries per liter
μeq/g	microequivalents per gram
μg	microgram
μg/g	micrograms per gram
μg/mL	micrograms per milliliter

1.0 INTRODUCTION

One major function of the Tank Waste Remediation System (TWRS) is to characterize wastes in support of waste management and disposal activities at the Hanford Site. Analytical data from sampling and analysis and other available information about a tank are compiled and maintained in a tank characterization report (TCR). This report and its appendixes serve as the TCR for single-shell tank 241-T-107.

The objectives of this report are as follows: 1) to use characterization data in response to technical issues associated with tank 241-T-107 waste, and 2) to provide a standard characterization of this waste in terms of a best-basis inventory estimate. Section 2.0 summarizes the response to technical issues, Section 3.0 shows the best-basis inventory estimate, and Section 4.0 provides recommendations about safety status and additional sampling needs. The appendixes contain supporting data and information. This report also supports the requirements of the *Hanford Federal Facility Agreement and Consent Order* (Ecology et al. 1996), Milestone M-44-05.

1.1 SCOPE

The characterization information in this report originated from sample analyses and known historical sources. Although only the results of recent sample events will be used to fulfill the requirements of the data quality objectives (DQOs), other information can be used to support (or question) conclusions derived from these results. Historical information (see Appendix A) includes surveillance information, records pertaining to waste transfers and tank operations, and expected tank contents derived from a process knowledge model.

Appendix B summarizes the recent sampling events listed in Table 1-1, sample data obtained before 1989, and the sampling results. The 1992 and 1993 core sampling effort was directed by the *Tank Waste Remediation System Tank Waste Characterization Plan* (Bell 1993). The results of the 1992 and 1993 sampling event were originally reported in *WHC 222-S and PNL-325 Single-Shell Tank Waste Characterization, 241-T-107 Cores 50, 51, and 52 - Data Package and Validation Summaries* (Svancara and Pool 1993) and *Statistical Characterization Report for Single-Shell Tank 241-T-107* (Jensen et al. 1994).

Before the 1992 and 1993 core sampling, a vapor sampling event was performed to address flammability issues. The results of the 1992 vapor sampling event were reported in Pingel (1992). The 1995 vapor sampling event satisfied the data requirements for this tank specified in the *Tank 241-T-107 Tank Characterization Plan* (Carpenter 1995). The results of the January 18, 1995, vapor sampling event were reported in *Tank 241-T-107 Headspace Gas and Vapor Characterization Results for Samples Collected in January 1995* (Huckaby and Bratzel 1995).

Appendix C reports on the statistical analysis and numerical manipulation of data used in issue resolution. Appendix D contains the evaluation to establish the best basis for the inventory estimate and the statistical analysis performed for this evaluation. Appendix E is a bibliography that resulted from an in-depth literature search of all known information sources applicable to tank 241-T-107 and its respective waste types. Most Appendix E reports can be found in the Tank Characterization Resource Center.

Table 1-1. Summary of Recent Sampling.

Sample/Date ¹	Phase	Location	Segmentation	% Recovery
Vapor sample (11/2/92)	Gas	Tank headspace	n/a	n/a
Core 50 (11/10/92)	Solid/ Liquid	Riser 2	1 ²	36
			1R	34
			2	94
			3	96
			4	67
Core 51 (2/18/93)	Solid/ Liquid	Riser 5	1	0
			2	64
			3U, 3L	100
			4U, 4L	100
Core 52 (3/10/93)	Solid/ Liquid	Riser 3	1	43
			2	56
			3U, 3L	95
			4	60
Vapor sample (1/18/95)	Gas	Tank headspace, riser 5, 5.5 m (18 ft) below top of riser	n/a	n/a

Notes:

- n/a = not applicable
- 1R = first segment was resampled
- U = upper half segment
- L = lower half segment

¹Dates are in the mm/dd/yy format.

²Segment was not used because it sat in the riser for more than 48 hours.

1.2 TANK BACKGROUND

Tank 241-T-107 is located in the 200 West Area T Tank Farm on the Hanford Site. It is the first tank in a three-tank cascade series. Tank 241-T-107 went into service in 1945 and received first cycle decontamination waste from B and T Plants. Supernate was transferred from the tank in 1951 in preparation to receive tributyl phosphate (TBP) from U Plant. Tributyl phosphate is also referred to as uranium recovery (UR) waste. During 1952 and 1953, TBP waste was transferred into tank 241-T-107.

In 1953 and 1954, the tank sent waste to tank 241-TX-118 and received unconcentrated ferrocyanide-scavenged TBP waste and flush water. Supernatant transfers out of the tank occurred in 1966. In 1967, tank 241-T-107 received plutonium-uranium extraction (PUREX) cladding waste from tank 241-C-102. In 1967, waste was transferred to tank 241-TY-103. During 1973, supernatant and flush water were transferred into tank 241-T-107, and supernatant was transferred out. Some waste was transferred to tank 241-T-101 in 1976. The tank was removed from service and declared inactive in 1976 when liquids were pumped from the tank in support of stabilization efforts (Agnew et al. 1996b). In 1983, additional salt well liquids were pumped to tank 241-AN-103. Interim stabilization was completed in May 1996.

Table 1-2 is a summary description of tank 241-T-107. The tank has an operating capacity of 2,010 kL (530 kgal) and presently contains an estimated 655 kL (173 kgal) of noncomplexed waste (Hanlon 1996). The tank was removed from the Ferrocyanide Watch List on September 4, 1996 (Kinzer 1996) and is not on other Watch Lists (Public Law 101-510).

Table 1-2. Description of Tank 241-T-107.

TANK DESCRIPTION	
Type	Single-shell
Constructed	1943 to 1944
In service	1945
Diameter	22.9 m (75.0 ft)
Operating depth	5.2 m (17 ft)
Capacity	2,010 kL (530 kgal)
Bottom shape	Dish
Ventilation	Passive
TANK STATUS	
Waste classification	Noncomplexed
Total waste volume	655 kL (173 kgal)
Supernatant volume	0 kL (0 kgal)
Saltcake volume	0 kL (0 kgal)
Sludge volume	655 kL (173 kgal)
Drainable interstitial liquid volume	83 kL (22 kgal)
Waste surface level (November 17, 1996)	172.5 cm (67.92 in.)
Temperature (September 1975 to November 1996)	11 °C (52 °F) to 33 °C (91 °F)
Integrity	Assumed leaker (1984)
Watch List	Ferrocyanide ¹ (1991 to 1996)
SAMPLING DATES	
Vapor samples	October 22, 1992 and January 18, 1995
Core samples	November 1992 to March 1993
SERVICE STATUS	
Declared inactive	1976
Interim stabilization	1996

Note:

¹Removed from Ferrocyanide Watch List on September 4, 1996.

2.0 RESPONSE TO TECHNICAL ISSUES

The technical issues that have been identified for tank 241-T-107 (Brown et al. 1996) are as follows:

Safety screening:

- Does the waste pose or contribute to any recognized potential safety problems?

Hazardous vapor safety screening:

- Does the vapor headspace exceed 25 percent of the lower flammability limit (LFL)? If so, what are the principal fuel components?
- Are compounds of technological significance present in the tank at a level that the industrial hygiene group shall be alerted to their presence so that adequate breathing zone monitoring can be accomplished and future activities in and around the tank can be performed in a safe manner?

Although the 1992 to 1993 core sampling event predated DQOs, results from the event have been used to address the first issue outlined in the safety screening DQO (Dukelow et al. 1995). Vapor screening was addressed according to the vapor DQO (Osborne et al. 1994) through the 1995 vapor sampling. Appendix B shows the analytical results from the core and vapor sampling events.

Tank 241-T-107 was on the Ferrocyanide Watch List and the 1992 and 1993 core samples were analyzed to support the resolution of the ferrocyanide safety issue. The ferrocyanide safety issue has been resolved; tank 241-T-107 was removed from the Ferrocyanide Watch List on September 4, 1996 (Kinzer 1996).

2.1 SAFETY SCREENING

The data needed to screen the waste in tank 241-T-107 for potential safety problems is documented in the *Tank Safety Screening Data Quality Objective* (Dukelow et al. 1995). These potential safety problems are exothermic conditions in the waste, flammable gases in the waste and/or tank headspace, and criticality conditions in the waste. Each condition is addressed separately below as applicable to the safety screening DQO. Because the 1992 and 1993 core sampling and analysis predate the DQO, however, sufficient data exist for comparisons to the requirements of the safety screening DQO.

2.1.1 Exothermic Conditions (Energetics)

The first requirement outlined in the safety screening DQO (Dukelow et al. 1995) is to ensure that exothermic constituents (organic or ferrocyanide) do not exist in tank 241-T-107 in sufficient quantities to pose a safety hazard. Because of this requirement, energetics in tank 241-T-107 waste were evaluated. The safety screening DQO required the waste sample profile be tested for energetics at the half segment, or every 24 cm (9.5 in.), to determine whether the energetics exceed the safety threshold limit. This requirement was met for three segments only. A majority of other segments were not broken down into halves because of low solids recovery.

The threshold limit for energetics is 480 joules per gram on a dry weight basis. Results obtained using differential scanning calorimetry (DSC) indicate that no exotherms were apparent for any analyses attributed to the tank waste (Svancara and Pool 1993). One exotherm was attributed to a plastic artifact that was mixed with the waste in the last segment of core 50. Further analyses of the plastic artifact with different carrier gases show the artifact to be anomalous and not representative tank waste (Svancara and Pool 1993). Because exotherms were not found in the waste, 95 percent confidence intervals were not calculated.

Historically, tank 241-T-107 waste is expected to have some exothermic properties because of the presence of ferrocyanide in the waste. However, recent studies (Babad et al. 1993 and Lilga et al. 1993, 1994, 1995, and 1996) and analytical data from other ferrocyanide tanks (for example, tanks 241-BY-104, 241-BY-106, 241-BY-108, and 241-C-108) have shown that a large degree of ferrocyanide decomposition probably occurs because of the combined effects of radiation, temperature, and pH in the harsh environments of the high-level radioactive waste tanks.

2.1.2 Flammable Gas

Vapor phase measurements, taken in the tank headspace on January 18, 1995, indicated the flammability of the headspace gases was below the safety screening threshold of 25 percent of the LFL (measurements of the lower explosive limit [LEL] are actually made; LEL is equivalent to LFL). Appendix B provides data from these vapor phase measurements and from vapor sampling on October 22, 1992.

2.1.3 Criticality

The safety screening DQO threshold limit is 1 g ²³⁹Pu per liter of waste. Assuming that all alpha is from ²³⁹Pu and using the overall bulk density of 1.51 g/mL, 1 g/L of ²³⁹Pu is equivalent to 41 μCi/g of alpha activity. Total alpha activity was analyzed in accordance with documents requiring analyses of composites instead of half segment or segment level analyses. The total alpha activities measured in the liquid and solid core composite samples

were well below the 41 $\mu\text{Ci/g}$ limit. The single largest result was 0.475 $\mu\text{Ci/g}$. The safety screening DQO requires the upper limit of the one-sided 95 percent confidence interval be calculated for each sample. However, because of the low results, no calculations were made for individual samples. A 95 percent confidence interval was calculated on the tank mean, yielding an upper limit of 0.930 $\mu\text{Ci/g}$. Therefore, criticality is not an issue for tank 241-T-107.

2.2 FERROCYANIDE ISSUES

The ferrocyanide safety issue has been resolved for tank 241-T-107; the tank was removed from the Ferrocyanide Watch List on September 4, 1996 (Kinzer 1996). A comparison is made in Section 2.5 between the 1992 and 1993 analytical results and the requirements of the ferrocyanide DQO (Meacham et al. 1995). This comparison is for information only because the requirements are no longer applicable.

2.3 VAPOR SCREENING

The 1995 vapor samples were taken to satisfy the requirements of the *Data Quality Objectives for Generic In-Tank Health and Safety Issue Resolution* (Osborne et al. 1994). The analyses required to meet the vapor DQO requirements were documented in Carpenter (1995). The vapor DQO addresses two problems: 1) potential flammable levels of gases and vapors generated or released in waste storage tank headspaces and, 2) the potential for worker hazards associated with the toxicity of constituents in any fugitive vapor emissions from these tanks. These problems are addressed below.

2.3.1 Flammable Gas

This is the same requirement as the safety screening flammability requirement except that the limit in Osborne et al. (1994) is 20 percent of the LFL instead of 25 percent. See Section 2.1.2 for a treatment of the flammability issue. All results from the January 18, 1995, vapor sampling were well below the DQO threshold, and it was determined that no organic or inorganic vapor posed a flammability hazard (Huckaby and Bratzel 1995).

2.3.2 Toxicity

To address the vapor DQO, Carpenter (1995) required the analysis of ammonia, carbon dioxide, carbon monoxide, hydrogen, methane, nitric oxide, nitrous oxide, nitrogen dioxide, tritium, and water vapor from samples of the tank headspace. Carpenter (1995) specified a threshold limit for each of these compounds except carbon dioxide, nitrous oxide, tritium, and water vapor. Aside from water and carbon dioxide, the most abundant waste constituents in the tank 241-T-107 headspace were ammonia (125 ppmv) and nitrous oxide

(41.5 ppmv). However, the concentrations of these species were below any limits listed in Carpenter (1995). Note that the concentrations of both ammonia and nitrous oxide exceeded the 25 ppmv limit of the current vapor DQO (Osborne and Buckley 1995).

In addition to the inorganic vapors, an analysis of organics was required from SUMMA¹ canisters and triple sorbent traps. The total organic vapor concentrations were found to be relatively low. The sum of quantitatively measured and estimated triple sorbent trap organic analyte concentrations was 1.4 mg/m³ (Huckaby and Bratzel 1995).

2.4 OTHER TECHNICAL ISSUES

A factor in assessing tank safety is the heat generation and temperature of the waste. Heat is generated in the tanks from radioactive decay. An estimate of the tank heat load based on radionuclide data from the 1992 and 1993 sample event is 779 W (2,660 Btu/hr), as shown in Table 2-1. Only the radionuclides present in detectable quantities were used in the calculation. This estimate agrees well with the heat load estimate based on tank headspace temperatures (708 W [2,416 Btu/hr]) (Kummerer 1995). The heat load estimate based on the tank process history was 37.4 W (128 Btu/hr) (Agnew et al. 1996a). All these estimates are quite low and are well below the 11,700 W (40,000 Btu/hr) limit that separates high- and low-heat load tanks (Smith 1986).

Table 2-1. Tank 241-T-107 Projected Heat Load.

Radionuclide	Curies	Watts
²⁴¹ Am	13.8	0.453
¹³⁷ Cs	12,200	57.6
^{239/240} Pu	148	4.51
⁹⁰ Sr	106,800	716
⁹⁹ Tc	50.0	0.0251
Total watts		779

¹SUMMA is a trademark of Moletrics, Inc., Cleveland, Ohio.

2.5 SUMMARY

This section summarizes the analytical results for the issues that apply to the tank 241-T-107 waste. Table 2-2 summarizes the results for the safety screening and vapor screening. Some uncertainty exists for total alpha analyses because only core composite data were available for evaluation. All safety screening primary analytes were well below threshold limits. As discussed previously, all the requirements of the vapor DQO (Osborne et al. 1994) were met. All analytical results were well below the ferrocyanide DQO thresholds.

Table 2-2. Summary of Safety Screening, Vapor Screening, and Ferrocyanide Evaluation Results.

Issue	Sub-Issue	Result
Safety screening	Energetics	No exotherms observed in tank waste samples.
	Flammable gas	All vapor measurements below 25 percent of LFL.
	Criticality	All analyses well below 41 $\mu\text{Ci/g}$ total alpha activity (composites).
Vapor screening	Flammability	All vapor measurements below 20 percent of LFL.
	Toxicity	All analytes were within the toxicity threshold limits of Osborne et al. (1994) ¹ .
Ferrocyanide ²	Energetics	No exotherms observed.
	Moisture	46.0 percent (Thermogravimetric Analysis [TGA] on segments) - well above limit.
	Nickel	292 ³ $\mu\text{g/g}$ (composites); far below 8,000 $\mu\text{g/g}$ limit.
	Cyanide	68.8 $\mu\text{g/g}$ (composites); far below 39,000 $\mu\text{g/g}$ limit.

Notes:

¹If the vapor results were evaluated against the current vapor DQO (Osborne and Buckley 1995), the ammonia and nitrous oxide concentrations would exceed toxicity limits.

²Tank 241-T-107 has been removed from the Ferrocyanide Watch List, and comparison to the ferrocyanide DQO is included for general information only.

³Acid digest results for nickel were used because of concerns that fusion results were contaminated from the nickel crucible using the fusion procedure.

This page intentionally left blank.

3.0 BEST-BASIS INVENTORY ESTIMATE

Information about the chemical and/or physical properties of tank wastes is used to perform safety analyses, engineering evaluations, and risk assessments associated with waste management activities, as well as to address regulatory issues. Waste management activities include overseeing tank farm operations and identifying, monitoring, and resolving safety issues associated with these operations and with the tank wastes. Disposal activities involve designing equipment, processes, and facilities for retrieving wastes and processing the wastes into a form that is suitable for long-term storage.

Chemical inventory information generally is derived using two approaches: 1) component inventories are estimated using the results of sample analyses, and 2) component inventories are predicted using a model based on process knowledge and historical information. The most recent model was developed by Los Alamos National Laboratory (Agnew et al. 1996a). Information derived from these two approaches is often inconsistent. An effort is underway to provide waste inventory estimates that will serve as standard characterization source terms for various waste management activities (Hodgson and LeClair 1996). As part of this effort, an evaluation of available chemical information for tank 241-T-107 was performed that included the following:

- Data from chemical analyses of three core samples collected from November 1992 through March 1993 (Svancara and Pool 1993).
- The solids composite inventory estimate generated from the Los Alamos National Laboratory model, *Hanford Tank Chemical and Radionuclide Inventories: HDW Model Rev. 3* (Agnew et al. 1996a), also referred to as the historical tank inventory estimate.

Results from this evaluation, which are detailed in Appendix D, support using sampling data as the basis for the best estimate inventory for tank 241-T-107 for the following reasons:

1. The inventory estimate generated by the Los Alamos National Laboratory model is based on a single defined waste stream and does not take into account any possible solids contributions from the TBP waste, the PUREX cladding waste, or the ion exchange waste received by the tank.
2. Comparisons demonstrate that neither the first cycle decontamination cycle BiPO₄ (1C) flowsheet nor the Hanford Defined Waste (HDW) model compare well with the analytical results. The historical tank inventory estimate needs to be reevaluated.

At present, the sample-based inventories derived from analytical data are the best-basis inventories. Tables 3-1 and 3-2 summarize the best-basis inventories for tank 241-T-107. A tank volume of 655 kL (173 kgal) and a density of 1.51 g/mL were used to calculate these inventories.

Table 3-1. Best-Basis Inventory Estimates for Nonradioactive Components in Tank 241-T-107. (2 sheets)

Analyte	Total Inventory (kg)	Best-Basis ¹ (S, M, or E)	Comment
Al	16,200	S	
Bi	11,100	S	
Ca	1,480	S	
Cl	541	S	
TIC as CO ₃	4,180	S	
Cr	343	S	
F	11,400	S	Based on water soluble portion only.
Fe	27,000	S	
K	231	S	
Mn	201	S	
Na	1.217E+05	S	
Ni	289	S	
NO ₂	11,700	S	
NO ₃	74,600	S	
Pb	636	S	
P as PO ₄	1.128E+05	S	
Si	6,000	S	
S as SO ₄	9,860	S	
Sr	852	S	

Table 3-1. Best-Basis Inventory Estimates for Nonradioactive Components in Tank 241-T-107. (2 sheets)

Analyte	Total Inventory (kg)	Best-Basis ¹ (S, M, or E)	Comment
TOC	1,680	S	
U _{TOTAL}	22,400	S	
Zr	112	S	

Notes:

TIC = total inorganic carbon

TOC = total organic carbon

¹Best-basis estimate based on S = Sampling, M = HDW model, and E = Engineering assessment.

Table 3-2. Best-Basis Inventory Estimates for Radioactive Components in Tank 241-T-107.

Analyte	Total Inventory (Ci)	Best-Basis ¹ (S, M, or E)	Comment
⁹⁰ Sr	1.068E+05	S	
⁹⁰ Y	1.068E+05	S	Based on ⁹⁰ Sr
⁹⁹ Tc	50	S	
¹³⁷ Cs	12,200	S	
¹³⁷ Ba	11,500	S	Based on ¹³⁷ Cs
^{239/240} Pu	148	S	
²⁴¹ Am	13.8	S	

Note:

¹Best-basis estimate based on S = sampling, M = HDW model, and E = engineering assessment.

This page intentionally left blank.

4.0 RECOMMENDATIONS

The January 18, 1995, vapor sampling event provided sufficient information to address the requirements of the vapor DQO (Osborne et al. 1994). No further vapor sampling efforts are necessary. The results satisfied the governing vapor DQO (Rev. 0). However, comparison to the current vapor DQO (Rev. 2) reveals that two analytes exceeded toxicity limits.

As discussed in Section 2.0, the 1992 and 1993 core sampling predated the existence of DQOs. Analytical results from this event were evaluated against the requirements of the safety screening DQO. All results were well within the safety notification limits. Although the ferrocyanide DQO was no longer applicable, an evaluation was also made between the DQO and the results; again, all requirements were satisfied. Although the sampling and analysis activities performed for tank 241-T-107 did not strictly meet the requirements for all applicable DQO documents (that is, some analyses were performed on core composites or whole segments instead of half segments), sufficient information is available to determine that notification limits would not be exceeded. Furthermore, a characterization best-basis inventory was developed for the tank contents.

Table 4-1 summarizes the status of Project Hanford Management Contractor (PHMC) TWRS Program review and acceptance of the sampling and analysis results reported in this TCR. Table 4-1, column 1 lists all DQO issues addressed by sampling and analysis. Column 2 indicates whether the requirements of the DQO were met by the sampling and analysis activities performed and is answered with a "yes" or a "no." Column 3 indicates concurrence and acceptance by the program in TWRS that is responsible for the DQO that the sampling and analysis activities performed adequately meet the needs of the DQO. A "yes" or "no" in column 3 indicates acceptance or disapproval of the sampling and analysis information presented in the TCR. If the results/information have not yet been reviewed, "N/R" is shown in the column. If the results/information have been reviewed, but acceptance or disapproval has not been decided, "N/D" is shown.

Table 4-1. Acceptance of Tank 241-T-107 Sampling and Analysis.

Issue	Sampling and Analysis Performed	Program ¹ Acceptance
Safety screening DQO	Yes	Yes
Vapor DQO	Yes	Yes

Note:

¹PHMC TWRS

Table 4-2 summarizes the status of PHMC TWRS Program review and acceptance of the evaluations and other characterization information contained in this report. The evaluations outlined in this report are the evaluation of worker hazards caused by contact with tank headspace vapors and the evaluation to determine whether the tank is safe, conditionally safe, or unsafe. Column 1 lists the evaluations performed. Columns 2 and 3 are in the same format as Table 4-1. The manner in which concurrence and acceptance are summarized is also the same as that in Table 4-1.

Table 4-2. Acceptance of Evaluation of Characterization Data and Information for Tank 241-T-107.

Issue	Evaluation Performed	Program ¹ Acceptance
Waste safety categorization (tank is safe)	Yes	Yes
Tank headspace vapors do not pose a safety concern	Yes	Yes

Note:

¹PHMC TWRS

One final comment regarding the safety screening DQO needs to be made. The one-sided confidence intervals that were used to determine whether ²³⁹Pu is below the DQO threshold were not calculated on each individual sample data because the data were not analyzed at the segment level, and the analytical results were well below the limits. Also it was not possible to check for contamination or dilution of the samples by the hydrostatic head fluid because water without a tracer was used for the head fluid (Valenzuela and Jensen 1994).

5.0 REFERENCES

- Agnew, S. F., J. Boyer, R. A. Corbin, T. B. Duran, J. R. Fitzpatrick, K. A. Jurgensen, T. P. Ortiz, and B. L. Young, 1996a, *Hanford Tank Chemical and Radionuclide Inventories: HDW Model Rev. 3*, LA-UR-96-858, Los Alamos National Laboratory, Los Alamos, New Mexico.
- Agnew, S. F., R. A. Corbin, T. B. Duran, K. A. Jurgensen, T. P. Ortiz, and B. L. Young, 1996b, *Waste Status and Transaction Record Summary for the Northwest Quadrant of the Hanford 200 East Area*, WHC-SD-WM-TI-669, Rev. 1, Westinghouse Hanford Company, Richland, Washington.
- Babad, H., R. J. Cash, J. E. Meacham, and B. C. Simpson, 1993, *The Role of Aging in Resolving the Ferrocyanide Safety Issue*, WHC-EP-0599, Westinghouse Hanford Company, Richland, Washington.
- Bell, K. E., 1993, *Tank Waste Remediation System Tank Waste Characterization Plan*, WHC-SD-WM-PLN-047, Rev. 1, Westinghouse Hanford Company, Richland, Washington.
- Brown, T. M., S. J. Eberlein, J. W. Hunt, and T. J. Kunthara, 1996, *Tank Waste Characterization Basis*, WHC-SD-WM-TA-164, Rev. 2, Westinghouse Hanford Company, Richland, Washington.
- Carpenter, B. C., 1995, *Tank 241-T-107 Tank Characterization Plan*, WHC-SD-WM-TP-295, Rev. 0A, Westinghouse Hanford Company, Richland, Washington.
- Dukelow, G. T., J. W. Hunt, H. Babad, and J. E. Meacham, 1995, *Tank Safety Screening Data Quality Objective*, WHC-SD-WM-SP-004, Rev. 2, Westinghouse Hanford Company, Richland, Washington.
- Ecology, EPA, and DOE, 1996, *Hanford Federal Facility Agreement and Consent Order*, as amended, Washington State Department of Ecology, U.S. Environmental Protection Agency, and U.S. Department of Energy, Olympia, Washington.
- Hanlon, B. M., 1996, *Waste Tank Summary Report for Month Ending October 31, 1996*, HNF-EP-0182-103, Lockheed Martin Hanford Corporation, Richland, Washington.
- Hodgson, K. M., and M. D. LeClair, 1996, *Work Plan for Defining a Standard Inventory Estimate for Wastes Stored in Hanford Site Underground Tanks*, WHC-SD-WM-WP-311, Rev. 1, Lockheed Martin Hanford Corporation, Richland Washington.

- Huckaby, J. L., and D. R. Bratzel, 1995, *Tank 241-T-107 Headspace Gas and Vapor Characterization Results for Samples Collected in January 1995*, WHC-SD-WM-ER-447, Rev. 1, Westinghouse Hanford Company, Richland, Washington.
- Jensen, L., R. D. Cromar, and S. R. Wilmarth, 1994, *Statistical Characterization Report for Single-Shell Tank 241-T-107*, WHC-SD-WM-TI-645, Rev. 0, Westinghouse Hanford Company, Richland, Washington.
- Kinzer, J., 1996, *Authorization to Remove the Remaining 14 Ferrocyanide Tanks 241-BY-103, 241-BY-104, 241-BY-105, 241-BY-106, 241-BY-107, 241-BY-108, 241-BY-110, 241-BY-111, 241-BY-112, 241-T-107, 241-TX-118, 241-TY-101, 241-TY-103 and 241-TY-104 from the "Watch List,"* (Letter 9602303 to A. L. Trego, September 4), U.S. Department of Energy, Richland Operations Office, Richland, Washington.
- Kummerer, M., 1995, *Topical Report on Heat Removal Characteristics of Waste Storage Tanks*, WHC-SD-WM-SARR-010, Rev. 1, Westinghouse Hanford Company, Richland, Washington.
- Lilga, M. A., M. R. Lumetta, and G. F. Schiefelbein, 1993, *Ferrocyanide Safety Project, Task 3 Ferrocyanide Aging Studies FY 1993 Annual Report*, PNL-8888, Pacific Northwest National Laboratory, Richland, Washington.
- Lilga, M. A., E. V. Alderson, D. J. Kowalski, M. R. Lumetta, and G. F. Schiefelbein, 1994, *Ferrocyanide Safety Project, Task 3 Ferrocyanide Aging Studies FY 1994 Annual Report*, PNL-10126, Pacific Northwest National Laboratory, Richland, Washington.
- Lilga, M. A., E. V. Alderson, R. T. Hallen, M. O. Hogan, T. L. Hubler, G. L. Jones, D. J. Kowalski, M. R. Lumetta, G. F. Schiefelbein, and M. R. Telander, 1995, *Ferrocyanide Safety Project: Ferrocyanide Aging Studies - FY 1995 Annual Report*, PNL-10713, Pacific Northwest National Laboratory, Richland, Washington.
- Lilga, M. A., R. T. Hallen, E. V. Alderson, M. O. Hogan, T. L. Hubler, G. L. Jones, D. J. Kowalski, M. R. Lumetta, W. F. Riemath, R. A. Romine, G. F. Schiefelbein, and M. R. Telander, 1996, *Ferrocyanide Safety Project: Ferrocyanide Aging Studies - Final Report*, PNNL-11211, Pacific Northwest National Laboratory, Richland, Washington.
- Meacham, J. E., R. J. Cash, B. A. Pulsipher, and G. Chen, 1995, *Data Requirements for the Ferrocyanide Safety Issue Developed Through the Data Quality Objective Process*, WHC-SD-WM-DQO-007, Rev. 2, Westinghouse Hanford Company, Richland, Washington.

- Osborne, J. W., J. L. Huckaby, T. P. Rudolph, E. R. Hewitt, D. D. Mahlum, J. Y. Young and C. M. Anderson, 1994, *Data Quality Objectives for Generic In-Tank Health and Safety Issue Resolution*, WHC-SD-WM-DQO-002, Rev. 0, Westinghouse Hanford Company, Richland, Washington.
- Osborne, J. W., and L. L. Buckley, 1995, *Data Quality Objectives for Tank Hazardous Vapor Safety Screening*, WHC-SD-WM-DQO-002, Rev. 2, Westinghouse Hanford Company, Richland, Washington.
- Pingel, L. A., 1992, *Waste Tank T-107 Vapor Sampling Results*, (internal memorandum 12240-SAS93-003 to G. L. Dukelow, November 2), Westinghouse Hanford Company, Richland, Washington.
- Public Law 101-510, 1990, "Safety Measures for Waste Tanks at Hanford Nuclear Reservation," Section 3137 of *National Defense Authorization Act for Fiscal Year 1991*.
- Smith, D. A., 1986, *Single-Shell Tank Isolation Safety Analysis Report*, WHC-SD-WM-SAR-006, Rev. 2, Westinghouse Hanford Company, Richland, Washington.
- Svancara, G. B., and K. N. Pool, 1993, *WHC 222-S and PNL-325 Single-Shell Tank Waste Characterization, 241-T-107 Cores 50, 51, and 52 - Data Package and Validation Summaries*, WHC-SD-WM-DP-042, Rev. 1A, Westinghouse Hanford Company, Richland, Washington.
- Valenzuela, B. D., and L. Jensen, 1994, *Tank Characterization Report for Single-Shell Tank 241-T-107*, WHC-SD-WM-ER-382, Rev. 0A, Westinghouse Hanford Company, Richland, Washington.

This page intentionally left blank.

APPENDIX A

HISTORICAL TANK INFORMATION

This page intentionally left blank.

APPENDIX A

HISTORICAL TANK INFORMATION

Appendix A describes tank 241-T-107 based on historical information. For this report, historical information includes any information about the fill history, waste types, surveillance, or modeling data about the tank. This information is necessary to provide a balanced assessment of the sampling and analytical results.

This appendix contains the following information:

- **Section A1:** Current tank status including the waste levels and the stabilization and isolation status.
- **Section A2:** Information about tank design.
- **Section A3:** Process knowledge of the tank, that is, the waste transfer history and the estimated contents of the tank based on modeling data.
- **Section A4:** Surveillance data for tank 241-T-107 including surface-level readings, temperatures, and a description of the waste surface based on photographs.
- **Section A5:** References for Appendix A.

Historical sampling results (results from samples obtained before 1989) are included in Appendix B.

A1.0 CURRENT TANK STATUS

As of October 31, 1996, tank 241-T-107 contained an estimated 655 kL (173 kgal) of noncomplexed waste. The waste volumes were estimated using photographs and surface-level measurements. Table A1-1 shows the volumes of waste phases found in the tank. In 1976, tank 241-T-107 was removed from service. It was declared an assumed leaker in 1984. The tank was interim stabilized in 1996; intrusion prevention has not been completed. The tank is passively ventilated, and all monitoring systems were in compliance with documented standards as of October 31, 1996. Tank 241-T-107 was removed from the Ferrocyamide Watch List on September 4, 1996 (Hanlon 1996).

Table A1-1. Tank Contents Status Summary¹.

Waste Type	kL (kgal)
Total waste	655 (173)
Supernatant	0
Sludge	655 (173)
Saltcake	0
Drainable interstitial liquid	83 (22)
Drainable liquid remaining	83 (22)
Pumpable liquid remaining	45 (12)

Note:

¹Hanlon (1996)

A2.0 TANK DESIGN AND BACKGROUND

Tank 241-T-107 was constructed during 1943 and 1944. It is one of twelve 2,010 kL (530 kgal) tanks in T Farm. The tanks were designed for nonboiling waste with a maximum fluid temperature of 104 °C (220 °F). A typical T Farm tank contains 9 to 11 risers ranging from 10 cm (4 in.) to 30 cm (12 in.) in diameter that provide surface-level access to the underground tank. Generally, there is one riser through the center of the tank dome and four or five each on opposite sides of the dome.

Tank 241-T-107 entered service in 1945 and is first in a three-tank cascading series. These tanks are connected by a 7.6-cm (3-in.) cascade line. The bottom center elevation of tank 241-T-107 is 193.5 m (635 ft) above sea level. The tank cascades to tank 241-T-108 at 193.0 m (633 ft), then to tank 241-T-109, which has a bottom center elevation at 192.3 m (631 ft). The cascade overflow height is approximately 4.78 m (188 in.) from the tank bottom and 60 cm (2 ft) below the top of the steel liner.

The single-shell tanks are constructed of 30-cm (1-ft)-thick reinforced concrete with a 6.4 mm (0.25 in.) mild carbon steel liner (ASTM A283 Grade C) on the bottom and sides and a 38-cm (1.25-ft)-thick-domed concrete top. The tanks have a dished bottom with a 1.2-m (4-ft) radius knuckle and a 5.2-m (17-ft) operating depth. The tanks are set on a reinforced concrete foundation.

A three-ply cotton fabric waterproofing was applied over the foundation and the steel tank. Four coats of primer paint were sprayed on all exposed interior tank surfaces. Tank ceiling domes were covered with three applications of magnesium zincfluorosilicate wash. Lead flashing was used to protect the joint where the steel liner meets the concrete dome. Asbestos gaskets were used to seal the access holes in the tank dome. The tanks were waterproofed on the sides and top with tar and a cement-like mixture. Each tank was covered with approximately 2.1 m (7 ft) of overburden.

Tank 241-T-107 has four process inlet nozzles and one cascade overflow inlet located approximately 4.8 m (188 in.) from the tank bottom (as measured at the tank wall). Figure A2-1 shows their locations.

Figure A2-2 shows a tank cross section, the approximate waste level, and a schematic of the tank equipment. Tank 241-T-107 has nine risers. Risers 2, 3, 5, 6, 7, and 8 are tentatively available for sampling (Lipnicki 1996). Risers 2, 3, 6, and 7 are all 30 cm (12 in.) in diameter. Risers 5 and 8 are 10 cm (4 in.) in diameter. Table A2-1 lists tank 241-T-107 risers showing their sizes and general use.

Figure A2-1. Riser Configuration for Tank 241-T-107.

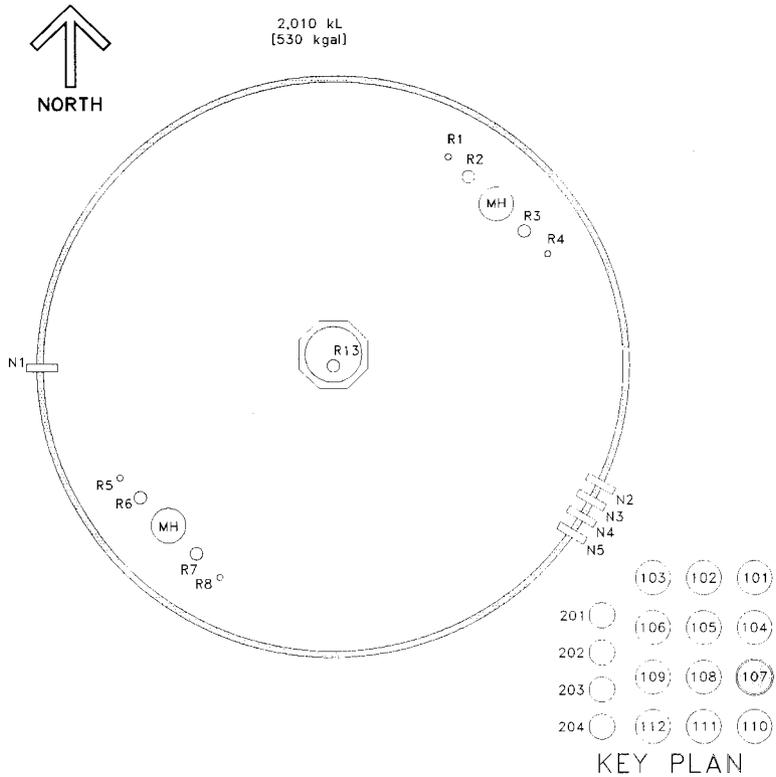


Figure A2-2. Tank 241-T-107 Cross Section and Schematic.

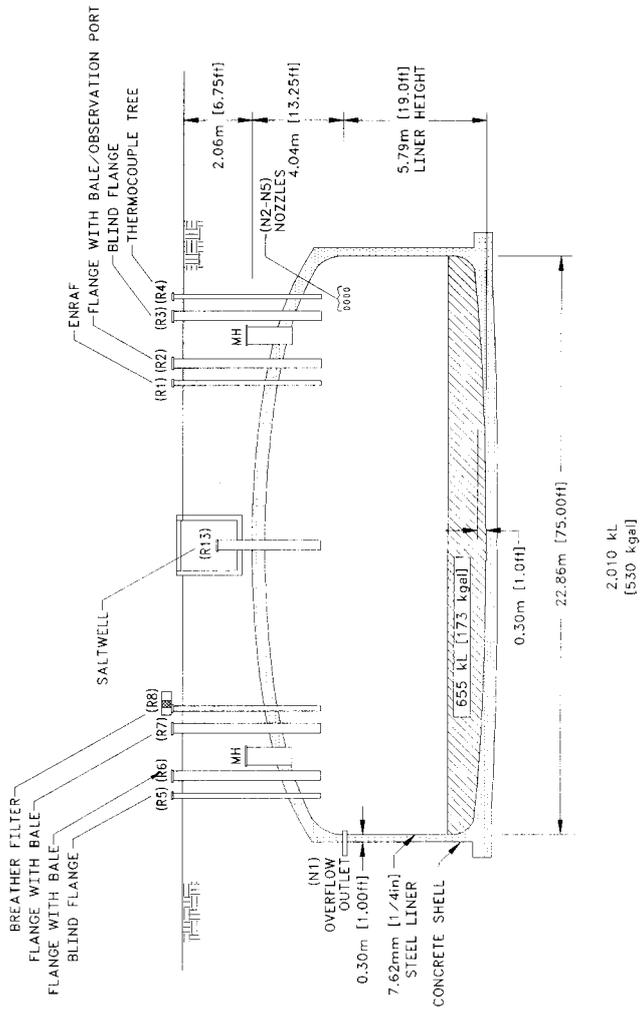


Table A2-1. Tank 241-T-107 Risers.^{1, 2, 3}

Number	Diameter		Description and Comments
	cm	in.	
R1	10	4	ENRAF ² gauge (installed June 1994) (previously contained an FIC gauge); (benchmarked December 11, 1986)
R2 ⁴	30	12	Flange with bale/observation port, spare
R3 ⁴	30	12	Blind flange
R4	10	4	Thermocouple tree
R5 ⁴	10	4	Blind flange; temperature vapor probe (September 22, 1994)
R6 ⁴	30	12	Flange with bale, spare
R7 ⁴	30	12	Flange with bale, spare
R8 ⁴	10	4	Breather filter
R13	30	12	Salt well riser; (benchmarked December 11, 1986)
N1	7.6	3	Cascade outlet nozzle
N2	7.6	3	Process inlet nozzle
N3	7.6	3	Process inlet nozzle
N4	7.6	3	Process inlet nozzle
N5	7.6	3	Process inlet nozzle

Notes:

FIC = Food Instrument Corporation

¹Alstad (1993)²Tran (1993)³Vitro Engineering Corporation (1988)⁴These risers are available for sampling according to Lipnicki (1996).²ENRAF is a trademark of the ENRAF Corporation, Houston, Texas.

A3.0 PROCESS KNOWLEDGE

The sections below: 1) provide information about the transfer history of tank 241-T-107; 2) describe the process wastes that made up the transfers; and 3) give an estimate of the current tank contents based on transfer history.

A3.1 WASTE TRANSFER HISTORY

Table A3-1 summarizes the waste transfer history of tank 241-T-107 (Agnew et al. 1996b). First-cycle decontamination (1C1) waste was added to tank 241-T-107 in the first quarter of 1945. This type of waste originated from the bismuth phosphate process performed at B and T Plants. The waste consists of by-product materials co-precipitated from a plutonium containing solution. The 1C1 waste stream also contained coating waste from the removal of the aluminum fuel element coating. First cycle decontamination waste was added continuously through the first quarter of 1946. Tank 241-T-107 was full in September 1945 and began to overflow into tank 241-T-108. The cascade was completely full by March 1946.

The tank remained undisturbed until 1951, then over half the supernatant waste was removed and sent to tank 241-TX-118. Tank 241-T-107 was reserved to receive TBP waste which came from the TBP uranium extraction process generated at U Plant. Tributyl phosphate waste (also called UR waste) was added to the tank from the end of 1952 until the first quarter of 1953. During this time, some waste cascaded to tank 241-T-108. In 1953, supernatant again was removed and sent to tank 241-TX-118 to feed the 200 West evaporator, leaving tank 241-T-107 slightly less than half-full with 908 kL (240 kgal) remaining in the tank.

From the last quarter of 1953 to the first quarter of 1954, tank 241-T-107 received flush water and unconcentrated, ferrocyanide-scavenged, TBP waste from tank 241-T-101. Between 1954 and late 1966, no addition or removal of waste was recorded, and the total waste volume remained constant. Supernatant tank waste was moved to tank 241-TX-118 from the second to fourth quarters of 1966, leaving 787 kL (208 kgal) of waste.

In 1967, tank 241-T-107 received cladding waste (CW) supernatant from tank 241-C-102. Cladding waste was produced at the PUREX plant from the processing of cold uranium. Approximately 1,040 kL (275 kgal) of CW waste, which included mostly liquids and a small amount of solids, were removed from tank 241-T-107 and transferred to tank 241-TY-103 in 1969.

In the beginning of 1973, tank 241-T-107 received flush water and supernatant waste from tank 241-BX-104. This was ion exchange waste (IX) from the cesium recovery process at B Plant. This waste, however, was immediately distributed to tanks 241-T-108 and 241-T-105 in the first and second quarters of 1973.

Supernatant was moved to tank 241-T-101 in 1976. Also in 1976, tank 241-T-107 was removed from service and designated inactive. In the third quarter of 1977, the solids level was adjusted to 568 kL (150 kgal), and the total volume was measured at 674 kL (178 kgal). In 1979, the integrity of the tank was questioned because of an anomalous activity noted in the dry wells or because of a noticeable drop in waste volume. Several level adjustments have occurred since. In February 1980, a new solids level of 632 kL (167 kgal) and a total volume of 674 kL (178 kgal) were recorded (Agnew et al. 1996b). A small amount of liquid was pumped to tank 241-AN-103 in the fourth quarter of 1983. Interim stabilization of tank 241-T-107 was completed in May 1996, and on May 31, 1996, a new solids level of 655 kL (173 kgal) was recorded (Hanlon 1996). Table A3-1 shows the major transfers of waste for tank 241-T-107.

Table A3-1. Tank 241-T-107 Major Transfers.^{1,2}

Transfer Source	Transfer Destination	Waste Type	Time Period	Estimated Waste Volume	
				kL	kgal
B and T plants	---	IC1	1945-1946	6,019	1,590
---	241-T-108	IC1	1945-1946	-4,005	-1,058
---	241-TX-118	SU	1951	-1,079	-285
U plant	---	TBP	1952-1953	4,804	1,269
---	241-T-108	SU	1952-1953	-3,725	-984
---	241-TX-118	SU	1953	-1,120	-296
241-T-101	---	SU (TBP)	1953-1954	1,037	274
Misc. sources	---	WTR	1954	106	28
---	241-TX-118	SU	1966	-1,207	-319
241-C-102	---	SU (CW)	1967	1,124	297
---	241-TY-103	SU	1969	-1,041	-275
Misc. sources	---	WTR	1973	49	13
241-BX-104	---	SU (IX)	1973	4,762	1,258
---	241-T-108	SU	1973	-2,449	-647
---	241-T-105	SU	1973	-1,711	-452
---	241-T-101	SU	1976	-833	-220
---	241-AN-103	SWLQW	1983	-19	-5

Notes:

SU = supernatant
 SWLQW = salt well liquid
 WTR = flush water

¹Agnew et al. (1996b)²Because only major transfers are listed, the sum of these transfers will not equal the current tank waste volume.

A3.2 HISTORICAL ESTIMATION OF TANK CONTENTS

The historical transfer data used for this estimate are from the following sources:

- *Waste Status and Transaction Record Summary for the Northwest Quadrant of the Hanford 200 West Area (WSTRS)* (Agnew et al. 1996b). The WSTRS is a tank-by-tank quarterly summary spreadsheet of waste transactions.
- *Hanford Tank Chemical and Radionuclide Inventories: HDW Model Rev. 3* (Agnew et al. 1996a). This document contains the HDW list, the Supernatant Mixing Model (SMM), and the Tank Layer Model (TLM).
- Historical Tank Content Estimate for the (Northeast, Northwest, Southeast, or Southwest) Quadrant of the Hanford 200 (East or West) Area (Brevick et al. 1996, 1997a, 1997b, 1997c). This set of four documents compiles and summarizes much of the process history, design, and technical information regarding the underground waste storage tanks in the 200 Areas.
- Tank Layer Model. The TLM defines the sludge and saltcake layers in each tank using waste composition and waste transfer information.
- Supernatant Mixing Model. This is a subroutine within the HDW model that calculates the volume and composition of certain supernatant blends and concentrates.

Using these records, the TLM defines the sludge and saltcake layers in each tank. The SMM uses information from the WSTRS and the TLM to describe the supernates and concentrates in each tank. Together the WSTRS, TLM, and SMM determine each tank's inventory estimate. These model predictions are considered estimates that require further evaluation using analytical data.

Various analytes are in the waste including sodium, nitrates, nitrites, sulfates, and hydroxides that are common to several waste streams. Over the years, the types and amounts of chemical compounds used in processes have varied to improve product recovery, and waste management practices (that is, evaporation and fission product recovery) have further confounded the waste matrix. Therefore, present concentrations may not parallel historical records.

Based on Agnew et al. (1996a), tank 241-T-107 contains two layers of 1C1 waste (total waste amount of 646 kL [171 kgal]). The bottom layer is predicted to be composed of 496 kL (131 kgal) of 1C1 waste, and the top layer contains 150 kL (40 kgal) of unknown waste which is assigned to 1C1.

Figure A3-1 is a graph of the estimated waste type and volume for the tank layers. Table A3-2 shows the historical tank inventory estimate for tank 241-T-107. The total

by Agnew et al. (1996a) differs slightly from the current estimate of 655 kL (173 kgal) because the waste volume estimate has been adjusted since.

Tank 241-T-107 was categorized as a ferrocyanide tank because it received waste from tank 241-T-101, a settling tank for the in-plant ferrocyanide scavenging process. The ferrocyanide scavenging method was performed on U Plant waste effluent, bismuth phosphate first cycle decontamination waste, and selected other wastes through an in-farm or in-plant scavenging process. In this process, sodium ferrocyanide and nickel were added to precipitate ¹³⁷Cs and other soluble radionuclides from the waste. Scavenging of ⁶⁰Co with Na₂S was also commonly done. It was possible tank 241-T-107 had greater than 1000 g-mol of ferrocyanide. Analytes that differentiate ferrocyanide waste from other wastes are elevated levels of nickel, calcium, and ¹³⁷Cs. However, the aging of the waste, exposure to radiation, and high pH within the tank are believed to have degraded the ferrocyanide (Lilga et al. 1996).

Figure A3-1. Tank 241-T-107 Tank Layer Model.

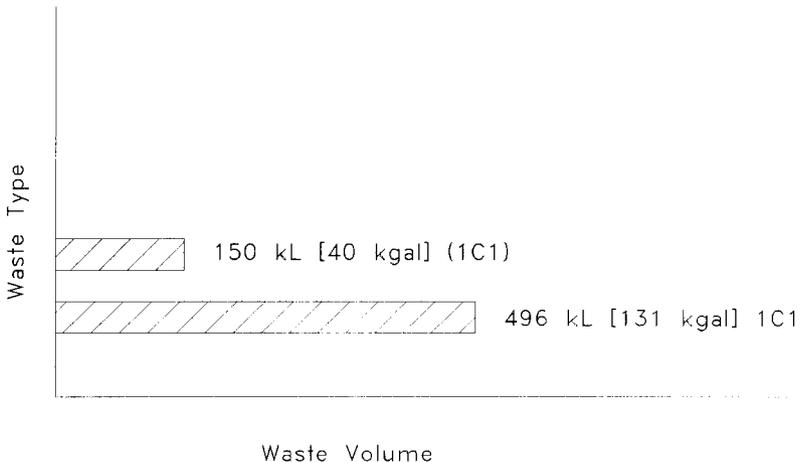


Table A3-2. Tank 241-T-107 Historical Tank Inventory Estimate.^{1,2} (2 sheets)

Total Inventory Estimate			
Physical Properties			
Total solid waste	8.70E+05 kg (180 kgal)		
Heat load	0.0374 kW (128 Btu/hr)		
Bulk density	1.28 (g/mL)		
Void fraction	--		
Water wt%	72.0		
Total organic carbon wt% carbon (wet)	0		
Chemical Constituents	<i>M</i>	$\mu\text{g/g}$	kg
Na ⁺	4.63	83,400	72,600
Al ³⁺	0.452	9,540	8,300
Fe ³⁺ (total Fe)	0.307	13,400	11,700
Cr ³⁺	0.00358	146	127
Bi ³⁺	0.0731	12,000	10,400
La ³⁺	0	0	0
Hg ²⁺	1.00E-04	15.8	13.7
Zr (as ZrO(OH) ₂)	0.00978	699	608
Pb ²⁺	0	0	0
Ni ²⁺	0.00110	50.7	44.1
Sr ²⁺	0	0	0
Mn ⁴⁺	0	0	0
Ca ²⁺	0.0717	2,250	1,960
K ⁺	0.00307	93.9	81.7
OH ⁻	2.49	33,200	28,800
NO ₃ ⁻	0.355	17,300	15,000
NO ₂ ⁻	0.170	6,130	5,330
CO ₃ ²⁻	0.0717	3,370	2,930
PO ₄ ³⁻	1.27	94,200	81,900
SO ₄ ²⁻	0.0424	3,190	2,780
Si (as SiO ₃ ²⁻)	0.0600	1,320	1,150

Table A3-2. Tank 241-T-107 Historical Tank Inventory Estimate.^{1,2} (2 sheets)

Total Inventory Estimate			
Chemical Constituents (Continued)	<i>M</i>	$\mu\text{g/g}$	kg
F ⁻	0.157	2,340	2,030
Cl ⁻	0.0141	391	340
citrate	0	0	0
EDTA ⁴⁻	0	0	0
HEDTA ³⁻	0	0	0
glycolate	0	0	0
acetate	0	0	0
oxalate	0	0	0
DBP	0	0	0
butanol	0	0	0
NH ₃	1.70E-04	2.27	1.97
Fe(CN) ₆ ⁴⁻	0	0	0
Radiological Constituents	Cl/L	$\mu\text{Ci/g}$	Cl
Pu	---	0.00570	0.0826 (kg)
U	5.43E-04 (<i>M</i>)	101 ($\mu\text{g/g}$)	88.0 (kg)
Cs	0.0116	9.07	7,890
Sr	1.02E-04	0.0803	69.8

Notes:

¹Agnew et al. (1996a)²These inventory predictions have not been validated and should be used with caution.

A4.0 SURVEILLANCE DATA

Tank 241-T-107 surveillance consists of surface-level measurements (liquid and solid), temperature monitoring inside the tank (waste and headspace), and leak detection well (dry well) monitoring for radioactivity outside the tank. Surveillance data provide the basis for determining tank integrity.

Liquid-level measurements may indicate whether the tank has a major leak. Solid surface-level measurements may indicate physical changes in and consistencies of the solid layers of a tank. Drywells located around the tank perimeter may show increased radioactivity caused by leaks. Tank 241-T-107 has no liquid observation well, but it does have three identified dry wells.

A4.1 SURFACE-LEVEL READINGS

Tank 241-T-107 is categorized as an assumed leaker. An automatic FIC gauge set in intrusion mode was used to monitor the surface level through riser 1 until June 1994. At that time, the FIC gauge was replaced with an ENRAFTM gauge. Manual readings are required daily if the ENRAFTM gauge fails or if the computer automated surveillance system readings are zero. The leak detection criteria for tank 241-T-107 are an increase of 5.1 cm (2.0 in.) or a decrease of 2.5 cm (1.0 in.) in intrusion mode from the baseline value. On September 30, 1995, there was an administrative solids level adjustment to 655 kL (173 kgal) with no supernatant. In March 1996, tank 241-T-107 met stabilization criteria; official declaration was made in May 1996. On May 31, 1996, a new solids level of 655 kL (173 kgal) was recorded (Hanlon 1996). On November 17, 1996, the automatic ENRAFTM reading was 172.6 cm (67.97 in.), and the manual ENRAFTM reading was 172.5 cm (67.92 in.). Figure A4-1 is a level history graph of the volume measurements.

A4.2 INTERNAL TANK TEMPERATURES

There are two thermocouple trees in tank 241-T-107, located in riser 4 (J type) and riser 5 (resistance temperature detector type). The data from the thermocouple tree in riser 5 closely matches the data from the thermocouple tree in riser 4. The thermocouple tree in riser 5 has 8 thermocouples. The thermocouple tree in riser 4 has 12 thermocouples to monitor the waste temperature. Thermocouple 1 is 36.6 cm (1.2 ft) from the tank bottom. Thermocouples 2 through 9 are spaced at 61-cm (2-ft) intervals above thermocouple 1. Thermocouples 9 through 11 are at 1.22-m (4-ft) intervals. The position of the 12th thermocouple probe is unknown; therefore, data from this thermocouple were not used (Brevick et al. 1995).

The temperature range from September 1975 to November 1996 was 11 °C (52 °F) to 33 °C (91 °F), and the average temperature was 18 °C (64 °F). On November 17, 1996, the average tank temperature was 19 °C (66 °F), the minimum was 18 °C (65 °F) from thermocouple 7 in riser 5, and the maximum was 19 °C (67 °F) from thermocouple 11 in riser 4. For plots of the thermocouple readings, refer to Brevick et al. (1995). Figure A4-2 shows a graph of the weekly high temperature.

Figure A4-1. Tank 241-T-107 Level History.

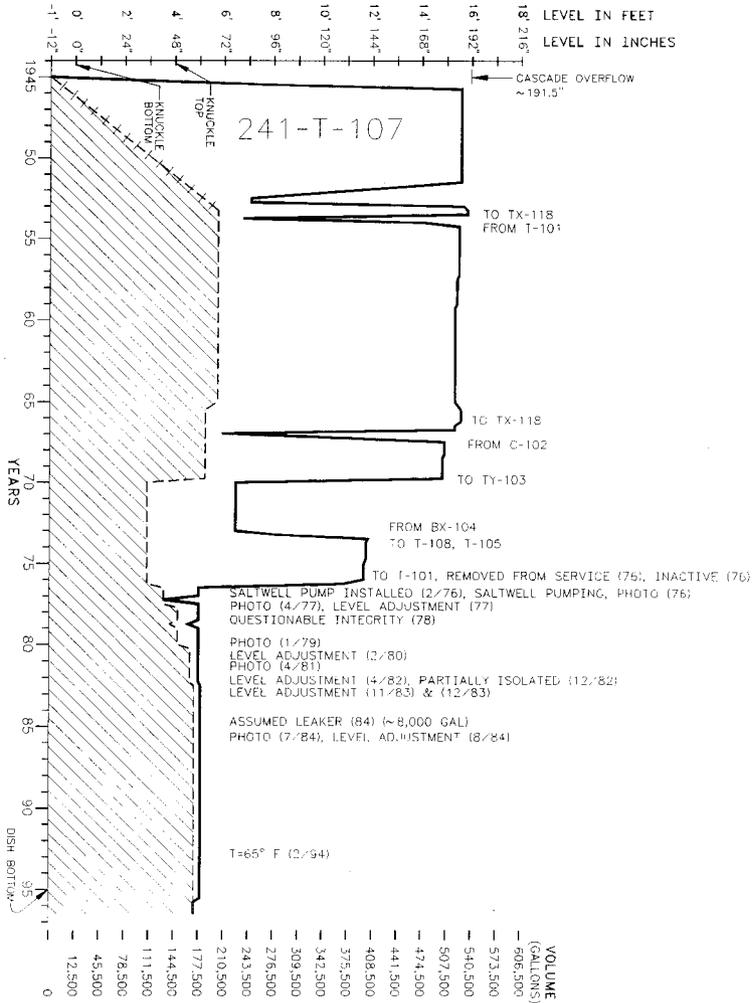
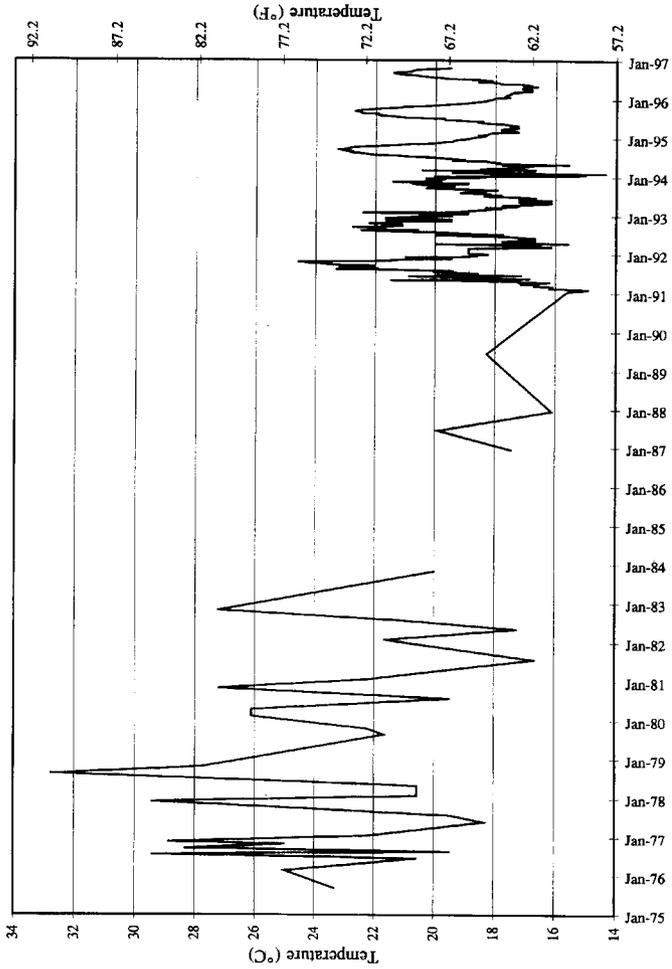


Figure A4-2. Tank 241-T-107 High Temperature Plot.



A4.3 TANK 241-T-107 PHOTOGRAPHS

For a photographic montage from July 12, 1984, refer to Brevick et al. (1995). Details within the tank are difficult to see because of the hazy quality of the photographs. The surface appears to be covered with a liquid except for a few places where solid mounds break through the liquid surface. An FIC probe, temperature probe, salt well screen, manhole, and some inlet nozzles have been identified and labeled. Because of salt well pumping and interim stabilization, the photographs do not represent current tank contents. In-tank videos were taken in September 1995 and May 1996 in support of interim stabilization following salt well pumping. The video shows an uneven surface, varying as much as 4 in. (Hanlon 1996). No supernate is visible on the surface.

A5.0 APPENDIX A REFERENCES

- Agnew, S. F., J. Boyer, R. A. Corbin, T. B. Duran, J. R. Fitzpatrick, K. A. Jurgensen, T. P. Ortiz, and B. L. Young, 1996a, *Hanford Tank Chemical and Radionuclide Inventories: HDW Model Rev. 3*, LA-UR-96-858, Los Alamos National Laboratory, Los Alamos, New Mexico.
- Agnew, S. F., R. A. Corbin, T. B. Duran, K. A. Jurgensen, T. P. Ortiz, and B. L. Young, 1996b, *Waste Status and Transaction Record Summary for the Northwest Quadrant of the Hanford 200 West Area*, WHC-SD-WM-TI-669, Rev. 1, Westinghouse Hanford Company, Richland, Washington.
- Alstad, A. T., 1993, *Riser Configuration Document for Single-Shell Waste Tanks*, WHC-SD-WM-RE-TI-053, Rev. 9, Westinghouse Hanford Company, Richland, Washington.
- Brevick, C. H., L. A. Gaddis, and W. W. Pickett, 1995, *Supporting Document for the Historical Tank Content Estimate for T Tank Farm - Volumes 1 and 2*, WHC-SD-WM-ER-320, Rev. 0, Westinghouse Hanford Company, Richland, Washington.
- Brevick, C. H., R. L. Newell, and J. W. Funk, 1996, *Historical Tank Content Estimate for the Northeast Quadrant of the Hanford 200 East Area*, WHC-SD-WM-ER-349, Rev. 1A, Westinghouse Hanford Company, Richland, Washington.
- Brevick, C. H., J. L. Stroup, and J. W. Funk, 1997a, *Historical Tank Content Estimate for the Southeast Quadrant of the Hanford 200 Areas*, HNF-SD-WM-ER-350, Rev. 1, Fluor Daniel Northwest, Richland, Washington.
- Brevick, C. H., J. L. Stroup, and J. W. Funk, 1997b, *Historical Tank Content Estimate for the Northwest Quadrant of the Hanford 200 West Area*, HNF-SD-WM-ER-351, Rev. 1, Fluor Daniel Northwest, Richland, Washington.
- Brevick, C. H., J. L. Stroup, and J. W. Funk, 1997c, *Historical Tank Content Estimate for the Southwest Quadrant of the Hanford 200 West Area*, HNF-SD-WM-ER-352, Rev. 1, Fluor Daniel Northwest, Richland, Washington.
- Hanlon, B. M., 1996, *Waste Tank Summary Report for Month Ending October 31, 1996*, HNF-EP-0182-103, Lockheed Martin Hanford Corporation, Richland, Washington.

- Lilga, M. A., R. T. Hallen, E. V. Aldersen, M. O. Hogan, T. L. Hubler, G. L. Jones, D. J. Kowalski, M. R. Lumetta, W. F. Riemath, R. A. Romine, G. F. Schiefelbein, and M. R. Telander, 1996, *Ferrocyanide Safety Project: Ferrocyanide Aging Studies - Final Report*, PNNL-11211, Pacific Northwest National Laboratory, Richland, Washington.
- Lipnicki, J., 1996, *Waste Tank Risers Available for Sampling*, WHC-SD-WM-TI-710, Rev. 3, Westinghouse Hanford Company, Richland, Washington.
- Tran, T. T., 1993, *Thermocouple Status Single-Shell & Double-Shell Waste Tanks*, WHC-SD-WM-TI-553, Rev. 0, Westinghouse Hanford Company, Richland, Washington.
- Vitro Engineering Corporation, 1988, *Piping Waste Tank Isolation 241-T-107*, Drawing H-73063, Rev. 3, Vitro Engineering Corporation, Richland, Washington.

APPENDIX B

SAMPLING OF TANK 241-T-107

This page intentionally left blank.

APPENDIX B

SAMPLING OF TANK 241-T-107

Appendix B provides sampling and analysis information for each known sampling event for tank 241-T-107 and an assessment of the 1992 and 1993 core sampling results.

- **Section B1:** Tank Sampling Overview
- **Section B2:** Sampling Events
- **Section B3:** Assessment of Characterization Results
- **Section B4:** References for Appendix B.

Future sampling of tank 241-T-107 will be appended to the above list.

B1.0 TANK SAMPLING OVERVIEW

Appendix B describes all known sampling events for tank 241-T-107, and provides the analytical results for each event. The sampling events listed include the 1992 and 1993 core sampling event, the 1992 and 1995 vapor sampling events, and the 1965, 1975, 1985, and 1989 historical supernatant sampling events.

Characterization of the solid portion of the tank waste is based on the 1992 and 1993 core sampling event. Core 50 was taken according to Winters et al. (1991); however, the core was characterized as directed in Bell (1993). Cores 51 and 52 were taken and analyzed as directed in Bell (1993). Because the sampling event predated DQOs, no DQO was applicable. An effort has been made to evaluate these results against the safety screening and ferrocyanide DQO requirements. The data package originally published the analytical results (Svancara and Pool 1993).

The tank headspace gases have been sampled twice. Before the November 1992 core sampling of tank 241-T-107, vapor samples were taken on October 1, 1992, for flammability issues. No DQOs were applicable because the event occurred before their existence. The results were reported in *Results from the Vapor Sampling of Waste Tank T-107* (Pingel 1992).

To support the safety screening DQO (Dukelow et al. 1995) and the vapor DQO (Osborne et al. 1994), additional vapor phase measurements were made on January 18, 1995. The vapor phase screening was taken for flammability and toxicity issues. The results were

reported in *Tank 241-T-107 Headspace Gas and Vapor Characterization Results for Samples Collected in January 1995* (Huckaby and Bratzel 1995).

Safety screening analyses include the following: total alpha to determine criticality, DSC to ascertain the fuel energy value, TGA to obtain the total moisture content, and bulk density. In addition, combustible gas meter readings in the tank headspace were performed to measure flammability. Table B1-1 summarizes sampling and analytical requirements from the safety screening, vapor, and ferrocyanide DQOs.

Table B1-1. Integrated Data Quality Objective Requirements for Tank 241-T-107.

Sampling Event	Applicable DQOs	Sampling Requirements	Analytical Requirements
1992 and 1993 core sampling	Safety screening ¹	Core samples from a minimum of two risers separated radially to the maximum extent possible.	<ul style="list-style-type: none"> ▶ Energetics ▶ Moisture content ▶ Total alpha ▶ Bulk density
	Ferrocyanide ^{1, 2}	A minimum of two cores. Efforts should be made to obtain good sample recovery and quarter segment analyses for primary analytes.	<ul style="list-style-type: none"> ▶ Energetics ▶ Moisture content ▶ Metals ▶ Anions ▶ Radionuclides
1992 and 1995 vapor phase measurements	Vapor ^{3, 4}	Measurement in a minimum of one location within tank headspace.	<ul style="list-style-type: none"> ▶ Vapor flammability ▶ Gases (NH₃, H₂, CH₄, H₂O, CO₂, CO, NO, NO₂, N₂O, tritium, and organics)

Notes:

¹DQO did not exist at the time of sampling.

²Removed from Ferrocyanide Watch List and no longer applicable.

³Osborne et al. (1994)

⁴DQO did not exist at the time of the 1992 vapor sampling.

Sampling data for tank 241-T-107 have been obtained for four historical liquid samples: September 1965, September 1975, March 1985, and August 1989. No information was available regarding sample handling or analysis for the samples; therefore, only analytical results and references are reported. The reason for the samplings were given as evaporator feed and mixing studies.

B2.0 SAMPLING EVENTS

B2.1 NOVEMBER 1992 - MARCH 1993 CORE SAMPLING EVENT

This section describes the core sampling and analysis event for tank 241-T-107 which occurred from November 1992 to March 1993.

B2.1.1 Description of Sampling Event

Tank 241-T-107 was push-mode core sampled through three risers during a period from November 5, 1992 to March 15, 1993. Approximately four segments were expected from each core sample (three full segments and one partial segment). Initially, two core samples were scheduled for tank 241-T-107, but because of poor sample recovery, a third core was taken (Silvers and Noonan 1993). Core 50 was obtained from riser 2, core 51 from riser 5, and core 52 from riser 3. The first core (core 50) was sampled on November 10, 1993; the second core sample (core 51) was completed on February 18, 1993; and the third core (core 52) on March 10, 1993. A chain-of-custody record was kept during the sampling event for each segment sampled and is available in the full data package (Svancara and Pool 1993).

The sampler is constructed of a stainless steel column 48-cm (19-in.) long, with a 2.2-cm (7/8-in.) inside diameter and a volume of 187 mL (0.05 gal). It is important to note that water, not normal paraffin hydrocarbons, was used as the hydrostatic head fluid for the tank 241-T-107 sampling event, potentially biasing the sample results. Water is a key component of the tank waste and, if the hydrostatic head fluid had leaked into the sample, the water content would appear much higher than it actually was. Lithium bromide, currently used as a tracer in the hydrostatic head fluid, was not used for the tank 241-T-107 sampling event. The 222-S Laboratory did not note any contamination of the sample by the hydrostatic head fluid; however, this type of contamination could be difficult to detect solely on a visual basis. Refer to the *Tank Characterization Reference Guide* (De Lorenzo et al. 1994) for more information on sampling.

B2.1.2 Sample Handling

The samples from tank 241-T-107 were transported in shipping casks to the 222-S Laboratory for analysis between November 10, 1992 and March 15, 1993. Further physical and radiochemical characterization was performed at the 325 Analytical Chemistry Laboratory. The 325 Analytical Chemistry Laboratory is operated by the Pacific Northwest National Laboratory and is located in the 300 Area of the Hanford Site.

Each segment remained in the cask until it was extruded from the sampler in the hot cell. The work performed on the samples was done remotely behind 60 cm (2 ft) of lead glass. Each sampler was placed in a horizontal position on the sample extruder, and the sample was removed using a piston positioned at the top of the sampler. The piston pushed the sample, bottom end first, onto a metal tray where solids and liquids were collected. A total of three core samples, averaging approximately three and a quarter segments per core, were taken from tank 241-T-107. If enough sample existed, each segment was divided into subsegments: upper (U) and lower (L). The drainable liquid from each core was collected and consolidated into a core drainable liquid composite sample, and the mass of the segment and the approximate length are recorded. From this information, the gross bulk densities were estimated. The sample volume was determined by measuring the length of the extruded sample and multiplying it by 9.85 mL/in. (The sampler had a volume of 187 mL for a sample length of 19 in.).

After the samples were extruded, they were photographed with the jar number and a color chart so that the description of each segment was consistent. The visual characteristics of the extruded samples and their mass and length were recorded in a log book. Special attention was placed on the sample volume, liquid/solid ratio, color, consistency, texture, and homogeneity of each segment. These notes helped provide qualitative descriptions for the cores. The written descriptions aided hot cell technicians in capturing the physical characteristics that a photograph cannot show, such as consistency and texture.

Because of the relatively low recovery, only three segments were divided into half-segments; the remaining segments were analyzed on a whole-segment basis. The material was homogenized and subsampled for laboratory analyses and archiving. Subsamples of each half-segment were recombined and subsampled for composite analyses and for shipment to Pacific Northwest National Laboratory for analysis. No solids composite was made for core 50 because of insufficient sample. Drainable liquid composites were made for all three cores. Table B2-1 summarizes the logbook and gives the subsampling scheme and sample descriptions.

Table B2-1. Tank 241-T-107 Subsampling Scheme and Sample Description.¹ (2 sheets)

Core	Seg	Percent Sampler Recovery	Sample Obtained		Comments
			Percent Solid	Percent Liquid	
50	1	36	72	28	22.87 g of very light to medium brown solids. Dark stripe down one side of the extruded solids. 8.75 g of opaque brown drainable liquid. Segment was resampled since it sat in the riser over 48 hours.
	1R	34	70	30	25.58 g of light brown solids homogeneous mixture. 10.89 g of opaque brown drainable liquid.
	2	94	100	0	194.45 g of solids. Sampler was under pressure. Solids were inhomogeneous and ranged from a light brown section, similar to Segment 1 except darker in color, to medium brown solids, to a dark brown section. No drainable liquids.
	3	96	5	95	Sample was recovered by holding the sampler vertical and tapping with a hammer. 8.52 g of dark brown solids were recovered. The solids were thick and homogeneous. 165 g of opaque brown drainable liquid; density of 0.97 g/mL.
	4	67	1	99	1.17 g piece of solids was recovered. 120.42 g of opaque brown drainable liquid; density of 0.97 g/mL.
51	1	0	0	0	Sampler was completely empty.
	2	64	40	60	64.48 g of dark brown solids. 87.30 g of opaque drainable liquid; density of 1.26 g/mL.
	3	100	100	0	215.66 g of dark brown solids. Solids appeared to be homogeneous. No drainable liquids.
	4	100	100	0	206.15 g of dark brown solids. Top 1 in. and bottom 6 in. appeared to have more fluids. No drainable liquids.

Table B2-1. Tank 241-T-107 Subsampling Scheme and Sample Description.¹ (2 sheets)

Core	Seg	Percent Sampler Recovery	Sample Obtained		Comments
			Percent Solid	Percent Liquid	
52	1	43	100	0	28.46 g of medium to dark gray solids. One side appeared to be dark gray, rest was light gray. No drainable liquids.
52	2	56	100	0	111.23 g of brown solids. Solids appeared wet.
	3	95	100	0	201.41 g of solids. Color ranged from light brown at bottom to dark brown at top. Solids were lumpy. No drainable liquids.
	4	60	3	97	4.25 g of light brown solids. 117.34 g of brown turbid drainable liquid; density of 1.12 g/mL.

Notes:

Seg = segment

¹Svancara and Pool (1993)

The general characteristics of tank 241-T-107 waste materials are as follows:

- Drainable liquids were brown in color and contained a large amount of suspended solids.
- Core samples ranged from light to dark brown with some medium to dark gray solids in the upper segments.
- The consistency of the solids ranged from a homogeneous slurry to a lumpy sludge. In all cases, the waste held its shape fairly well.
- Poor recovery from the first riser prompted the sampling of a third riser.

The next step in the sample preparation process was the distribution of aliquots for the various analytical procedures. The unhomogenized (direct) samples were obtained by pushing a small open metal tube into the segment. These were used for particle size analysis and volatile organic analysis. Subsequent homogenization of the segments was performed. This was done in an apparatus called a stomacher, a machine with paddles. A bag containing the sample was placed in the stomacher, and the samples were homogenized by a process similar to kneading bread. A majority of the analyses were performed on the homogenized samples. By mixing equal portions of each homogenized segment together, it is believed that a representative composite for each core was obtained. When homogenization was completed and aliquots were removed for analysis, the remaining sample was archived and stored at the 222-S Laboratory. Segments, composite samples, and subsamples were often divided into different aliquots to satisfy sample analysis requirements.

B2.1.3 Sample Analysis

The analyses performed on the core samples were those required under Module B2 of Bell (1993). Quality control tests included performing the analyses in duplicate and the use of standards, spikes, and blanks. To verify analyte recoveries resulting from separation techniques, laboratory control samples, carriers, spikes, tracers, and surrogates were analyzed concurrently with the samples.

Sample preparation procedures were conducted to optimize the recovery of each analyte of interest from the tank waste. Water digestion, acid digestion, and potassium hydroxide fusion were used to extract metals and several radioisotopes from solid samples. In some cases, digestions were performed on liquid samples to improve analytical matrices. Table B2-2 describes many separations that were specific to a particular analysis.

In some cases, no sample preparation was necessary or desired. Direct analyses were performed on the sample matrix with little or no sample preparation. Several direct analyses were performed relating to the physical or energetic properties of the waste: density, TGA, DSC, and gravimetric weight percent water.

Water digestion (leach) analyses were performed after the sample matrix was digested in distilled/deionized water; then the water was analyzed for soluble analytes. The soluble anions were determined by ion chromatography (IC). The primary anions analyzed in this manner were fluoride, chloride, nitrate, nitrite, phosphate, and sulfate. In addition, TOC, total inorganic carbon (TIC), free cyanide, pH, and ammonia were also analyzed from water digestion samples by various analytical methods. Note that IC assays used a 1:100 sample:water dilution, where pH measurements used a 1:1 sample:water ratio. Selected radionuclides were measured on some water digestion samples to determine the type and number of soluble radionuclides. Inductively coupled plasma (ICP) was performed on some water digestion samples to determine the amount of soluble metal cations. Nitrite and

Table B2-2. Solid Samples Preparation Summary.

Direct	Fusion Dissolution	Water Leach	Acid Digestion
Segment or Subsegment			
- Persulfate oxidation (TC/TOC/TIC) - TGA - DSC - Spectrometry (CN) - Gravimetry - Bulk density	- ICP - GEA - BPC (⁹⁰ Sr) - LF - APC (total alpha Pu) - Alpha spectrometry - Mass spectrometry	- pH	- ICP
Composite			
- Persulfate oxidation (TC/TOC/TIC) - SVOA - Spectrometry (CN) - Gravimetry - CVAA (Hg) - Bulk density - Rheology - Physical properties	- ICP - GEA - AA - APC - BPC (Total beta, ⁹⁰ Sr) - Alpha spectrometry - Mass spectrometry - Liq. scin. (⁹⁹ Tc) - LF	- IC - ICP - GEA - Distillation (NH ₃) - Spectrophotometry (Cr ⁶⁺ , NO ₂) - Spectrometry (CN) - Furnace oxidation (TOC/TIC) - APC - BPC (Total beta) - Liq. scin. (¹⁴ C, ³ H) - pH - Wt% residual solids - Residual solids	- ICP - AA

Notes:

AA	=	atomic absorption
APC	=	alpha proportional counting
BPC	=	beta proportional counting
CVAA	=	cold vapor atomic absorption
GEA	=	gamma energy analysis
LF	=	laser fluorimetry
Liq. scin.	=	liquid scintillation
SVOA	=	semivolatile organic analysis
TC	=	total carbon

chromium (VI) from water digestions were analyzed by spectrophotometry. A total alpha and total beta count were performed on the water digestion samples as well. Gamma energy analysis was also performed on water digestion samples to detect water soluble radionuclides. In many cases, these analytes were below the detection limits for the water digestion samples, suggesting that many analytes are not water soluble.

For acid digestion preparation, the sample was dissolved in a mixture of hydrochloric and nitric acids. This preparation brings most insoluble metals into solution with a minimum amount of dilution and was considered best for the detection of trace elements and some major metals. Some elements occur in the tank in relatively large quantities and were referred to as major metals. These properties are the reason that acid digestion is generally used as the sample preparation for homogenization tests. The analyses performed on this preparation were ICP and AA. (The AA analysis used nitric acid only.) Analyzing an acid digestion solution using ICP analysis detected elemental compositions within the waste, especially trace and major metals.

Experience with Hanford tank waste matrices has shown that acid digestion does not always provide complete solubilization and that a more rigorous dissolution preparation (that is, fusion) may be necessary to get adequate quantitation. Analyses that were performed on fusion-prepared samples included ICP, AA, and LF for metals; and GEA, APC, alpha spectrometry, BPC, liquid scintillation, and mass spectrometry for radionuclides. Fusion dissolution analyses were performed on the sample matrix after fusion with potassium hydroxide in a nickel crucible, then dissolution in acid. This preparation dissolves the entire sample, whereas other sample preparation procedures may not completely dissolve the sample matrix. One significant disadvantage of fusion preparation is that large amounts of potassium hydroxide are required to bring a sample into solution, which means a large dilution is involved. Because of this high dilution factor, trace elements are less likely to be correctly quantified, if they are detected at all. Another limitation of the preparation method is if the sample contains substantial quantities of potassium or nickel, these analytes will not be quantifiable, because the procedure uses potassium hydroxide and a nickel crucible. (This limitation can be overcome using alternate preparation methods, if potassium or nickel are analytes critical to interpretation of the data.) Elements that occur in abundance (major metals) or are highly insoluble are likely to be detected better by the fusion results than by any other sample preparation.

Generally, fusion dissolution is the preferred method of analyzing radionuclide content, with the exception of ^{14}C , ^{129}I , and ^3H (tritium). However, the sample preparation specified in the test instructions for ^{14}C (water digestion) is probably not the best for the high-level waste matrices. Difficulty with dissolving the sample with a water leach and volatility associated with a fusion preparation will potentially bias the ^{14}C results low for both sample preparation types, if they are associated with the water insoluble solid materials. Similar difficulties are encountered for other radionuclides. However, none of these analytes are expected to be significant contributors to the radionuclide content of the waste.

The major analytes detected during the fusion ICP analysis for tank 241-T-107 were aluminum, bismuth, iron, sodium, phosphorus, silicon, and sulfur. In the case of these elements, the fusion result was the preferred method of analysis, because it was believed to provide more complete dissolution of the waste and a more complete quantitation of analytes. Comparing these results with IC results can provide insight to the solubility characteristics of the waste. Some of the primary radionuclides that were measured using fusion preparation are ^{241}Am , $^{239/240}\text{Pu}$, ^{90}Sr , ^{137}Cs , and ^{99}Tc . A total alpha and total beta count were performed on the fusion dissolution samples as well.

All reported analyses were performed in accordance with approved laboratory procedures. Tables B2-3 and B2-4 list the procedure numbers and applicable analyses.

Table B2-3. Analytical Methods For Organic and Physical Analyses.

Analysis	Method	Procedure Number
SVOA	Gas chromatography/mass spectrometry (GC/MS)	PNL-ALO-345
TGA	TGA	LA-560-112
Percent water	Gravimetry	LA-564-101
DSC	DSC	LA-514-113
Density	Bulk density	LA-560-101
Specific gravity	Specific gravity	LA-510-112
Rheology	Rheology	PNL-ALO-501
Particle size	Particle size	T044-A-01712F

Table B2-4. Analytical Methods for Chemical and Radionuclide Analyses.

Analyte	Method	Procedure Number
Cs	AA spectroscopy	LA-505-121
Hg	CVAA	LA-325-104
Total metals	ICP	LA-505-151
F ⁻ , Cl ⁻ , NO ₃ ⁻ , NO ₂ ⁻ , PO ₄ ³⁻ , SO ₄ ²⁻	IC	LA-533-105
CN ⁻	Distillation/spectrometric analysis	LA-695-102
NO ₂ ⁻	Spectrophotometry	LA-645-001
Cr ⁶⁺	Spectrophotometry	LA-265-101
NH ₃	Distillation/Kjeldahl	LA-634-102
H ⁺	pH	LA-212-102 LA-212-103
U	LF	LA-925-106
Total alpha Total beta	Proportional counting	LA-508-104 LA-508-101 LA-508-114
²³⁸ Pu, ^{238,240} Pu, ²⁴¹ Am	Alpha spectrometry	LA-503-156 LA-508-051
⁹⁰ Sr	BPC	LA-220-101
⁹⁹ Tc ¹⁴ C ³ H	Liquid scintillation	LA-438-101 LA-348-104 LA-218-114
¹⁵⁴ Eu, ¹⁵⁵ Eu, ²⁴¹ Am, ¹³⁷ Cs, ⁶⁰ Co	GEA	LA-548-121 LA-508-052 LA-504-101
¹²⁹ I	Low energy gamma analysis	LA-378-103
Pu isotopic	Fusion mass spectrometry	PNL-ALO-423 PNL-MA-597
U isotopic	Mass spectrometry	PNL-MA-597 PNL-ALO-445
TOC	TOC	LA-344-105 PNL-ALO-381
CO ₃ /C	TIC	LA-622-102 PNL-ALO-381

B2.1.4 Analytical Results

This section summarizes the sampling and analytical results associated with the 1992 and 1993 sampling and analysis of tank 241-T-107. Table B2-5 describes the chemical, physical, and thermodynamic results. All results are documented in the validated data package (Svancara and Pool 1993).

Table B2-5. Analytical Presentation Tables.

Analysis	Table Number
Physical properties summary	B2-6
Particle size data	B2-7
Metals by graphite atomic absorption	B2-8
Metals by CVAA spectroscopy	B2-9
Metals by ICP spectroscopy	B2-10 through B2-40
Hexavalent chromium by spectrophotometry	B2-41
Uranium by LF	B2-42
Anions by IC	B2-43 through B2-48
Ammonia by distillation	B2-49
Nitrite by spectrophotometry	B2-50
Cyanide by distillation/spectrometry	B2-51
SVOA	B2-52
Analyses for TC/TIC/TOC	B2-53 through B2-55
Radionuclides by APC	B2-56
Radionuclides by BPC	B2-57 and B2-58
Radionuclides by alpha spectrometry	B2-59 through B2-61
Radionuclides by mass spectrometry	B2-62 through B2-70
Radionuclides by GEA	B2-71 through B2-82
Radionuclides by liquid scintillation	B2-83 through B2-85
Analyses for physical properties	B2-86 through B2-93
Analyses for rheological properties	B2-94 through B2-96
1992 vapor sampling results	B2-97
1995 vapor sampling results	B2-98
Historical sampling results	B2-99 through B2-102

A complete validation was performed on the data. Many quality control (QC) and quality assurance parameters were investigated during the validation including standard recoveries, spike recoveries, duplicate analyses, and blanks. For complete data validation information, refer to Svancara and Pool (1993). For more information about the QC investigation and a summary of the data validation findings, refer to Section B3.2.

The following subsections discuss the methods used in analyzing the core samples. Because of the large size of the data set, all discussion of the analytical procedures is provided first, followed by the data tables. For most analytes (except for some physical and rheological measurements), the data tables consist of six columns. Column 1 shows the sample number. Column 2 delineates the core and/or segment from which the samples were derived. An entry of "50:3" means core 50, segment 3. Column 3 lists the sample portion from which the aliquots were taken. For ICP analytes, no distinction was made between the duplicate analyses for some segments which were performed for homogenization tests. The final three columns display the primary and duplicate analytical values and a mean for each sample/duplicate pair.

B2.1.4.1 Inorganic Analyses. The characterization plan (Bell 1993) required that anions and metals be analyzed. Metals were determined following three different sample treatments: 1) water extraction, 2) acid digestion, and 3) potassium hydroxide fusion. The anions were prepared by water extraction. Drainable liquid composite samples were analyzed directly.

Atomic Absorption Spectroscopy. Atomic absorption analyses for cesium were performed according to procedure LA-505-121 on cores 51 and 52 composites after fusion, and on the core 51 composite after an acid digestion. Table B2-8 shows the results.

Cold Vapor Atomic Absorption Spectroscopy. Mercury was analyzed on the solids composites by CVAA spectroscopy according to procedure LA-325-104. Table B2-9 shows the results.

Inductively Coupled Plasma Spectroscopy. The following analytes were evaluated by ICP according to procedure LA-505-151: aluminum, antimony, arsenic, barium, beryllium, bismuth, boron, cadmium, calcium, cerium, chromium, cobalt, copper, iron, lanthanum, lead, magnesium, manganese, nickel, potassium, phosphorus, selenium, silicon, silver, sodium, strontium, sulfur, zinc, and zirconium. Aluminum, bismuth, iron, phosphorus, and sodium were the most abundant metals in tank 241-T-107. The core composites were analyzed after acid, fusion, and water digestions. The two fusion digest samples of core 52 originated from the same core composite aliquot before fusion preparation. Segment analyses were performed after acid and fusion digestions. Tables B2-10 through B2-40 show the results.

Chromium (VI) by Spectrophotometric Analysis. Analyses for chromium (VI) were performed by spectrophotometry on composite samples which had been water leached. The analyses were performed according to procedure LA-265-101. Table B2-41 shows the results of this analysis.

Laser Fluorimetry. Total uranium concentrations were measured in the fusion composite samples using LF. The analyses were performed according to procedure LA-925-106. Table B2-42 shows the results of this analysis.

Ion Chromatography. The following anions were determined by IC according to procedure LA-533-105: chloride, fluoride, nitrate, nitrite, phosphate, and sulfate. All IC analyses were performed on core composites. The solids analyses were performed after water digestions. The two water digest samples for the solids composite of core 52 originated from the same core composite aliquot after water digestion. Tables B2-43 through B2-48 show the analytical results.

Ammonia by Distillation. Ammonia analysis was performed by procedure LA-634-102 on the water-leached solids composites. Table B2-49 shows the results.

Nitrite by Spectrophotometric Analysis. Nitrite was determined according to procedure LA-645-001. This analysis was performed on water digested core composites and the drainable liquids. Table B2-50 shows the results.

Distillation/Spectrometric Analysis. Cyanide was determined according to procedure LA-695-102. Analyses were performed directly on the segments and composites. In addition, water digestions of the core composites were made. Table B2-51 shows the results.

B2.1.4.2 Organic Analyses

Semivolatile Organic Compounds. Semivolatile organic compounds were determined according to procedure PNL-ALO-345. Table B2-52A shows the detected results, and Table B2-52B shows the analytes with all nondetected values.

B2.1.4.3 Carbon

Total Carbon. Total carbon was analyzed by persulfate oxidation directly on the segments and solids composites. Table B2-53 shows the results.

Total Organic Carbon. Total organic carbon was determined by persulfate oxidation using procedures LA-344-105 and PNL-ALO-381. Analyses were performed directly on the segments and core composites. Total organic carbon was measured by furnace oxidation on water leaches of the core composites. Table B2-54 shows TOC results.

Total Inorganic Carbon. Total inorganic carbon was determined by coulometry measurements of the CO₂ evolved following sample acidification, as established in procedures LA-344-102 and PNL-ALO-381. Analyses were performed directly on the segments and core composites. Core composite TIC concentrations were also measured by furnace oxidation after a water leach. Table B2-55 shows the TIC results.

B2.1.4.4 Radiochemical Analyses

Alpha Proportional Counting. Alpha proportional counting was used to determine total alpha activity according to procedures LA-508-101 and LA-508-104. Analyses were performed on fused and water digested core composites. Table B2-56 shows the sample results.

Beta Proportional Counting. Beta proportional counting was used to determine total beta activity and ⁹⁰Sr activity according to procedures LA-508-114, LA-508-101, and LA-220-101, respectively. Analyses were performed both on core composite samples which had been fused and composites which had been water leached. Tables B2-57 and B2-58 show the results.

Alpha Energy Spectrometry. The following were evaluated on the core composites by alpha energy spectrometry according to procedures LA-503-156 and LA-508-051: ²⁴¹Am, ²³⁸Pu, and ^{239/240}Pu. Tables B2-59 through B2-61 show the sample results.

Mass Spectrometry. Thermal ionization mass spectrometry was used to determine the presence of all isotopes of Pu according to procedures PNL-ALO-423 and PNL-MA-597 and all isotopes of U according to procedures PNL-MA-597 and PNL-ALO-445. Uranium and plutonium values were consistent with typical fuel burn up. Because of the low ²³⁸Pu concentration and the high uranium concentration in the samples, the uranium contamination in the purified Pu fraction interfered with the mass spectrometric determination of ²³⁸Pu. Therefore, ²³⁸Pu was determined by alpha energy spectrometry. Because of the small quantity of Pu in the samples and the low isotopic abundance of ²⁴¹Pu and ²⁴²Pu, values reported for these isotopes are best estimates only. Tables B2-62 through B2-70 show the mass spectrometry results.

Gamma Energy Analysis. The activities of the following radionuclides were determined by GEA according to procedures LA-548-121 and LA-508-052: ²⁴¹Am, ¹⁴⁴Ce/Pr, ¹³⁴Cs, ¹³⁷Cs, ⁶⁰Co, ¹⁵⁴Eu, ¹⁵³Eu, ⁴⁰K, ¹⁰³Ru, ¹⁰⁶Ru/Rh, and ²²⁸Th. The activity of ¹²⁹I was determined on the liquids by low energy gamma analysis according to procedure LA-378-103. Composites samples were prepared for analysis using both fusion and water digestion. Segment samples were analyzed only after fusion. Tables B2-71 through B2-82 show the GEA results.

Liquid Scintillation. Tritium, ¹⁴C, and ⁹⁹Tc were analyzed by liquid scintillation according to procedures LA-218-114, LA-348-104, and LA-438-101, respectively. Solids composites were measured after water digestion except for ⁹⁹Tc, which was measured after fusion. Tables B2-83 through B2-85 show the sample results.

B2.1.4.5 Physical Analyses. At the time of the sampling and analysis of tank 241-T-107, no DQO existed to define the scope of the analyses. However, several analytes relating specifically to physical properties were determined to be of interest to the waste characterization program; they are summarized here. The physical characteristics of tank waste are required to develop and provide a basis for validation of equipment testing using design criteria and simulated waste.

Density/Specific Gravity. Upon extrusion, a rough density calculation was made for each segment from both cores by dividing the mass recovered for that segment by its volume. Table B2-87 shows the results. In addition, more precise analytical density determinations were performed for all segments according to procedure LA-560-102. Table B2-86 shows these values. Specific gravity measurements were made on the liquid composites according to procedure LA-510-112. Table B2-88 shows the results.

pH. Sample pH was measured on solid segment samples and core composite samples, drainable liquid core composite samples, and on the field and hot cell blanks according to procedures LA-212-103 for solids and LA-212-102 for liquids. If any pH was greater than 12.5, a hydroxide analysis was to be completed. All sample pH measurements were less than 12. Table B2-89 shows the results.

Physical Measurements. Physical testing was performed on unhomogenized material from one segment of core 50. The testing included density (centrifuged supernate and solids densities) and settling behavior (volume percent centrifuged solids, volume percent settled solids, weight percent centrifuged solids, weight percent solids, weight percent dissolved solids, and weight percent undissolved solids). Table B2-6 shows the results.

Table B2-6 data indicate the as-received sample did not settle, but a substantial amount of liquid was associated with the sample, as was observed by the volume percent and weight percent centrifuged solids. This conclusion is supported by the weight percent solids data. A two-fold decrease in the volume percent of settled and centrifuged solids, between each dilution and linear decrease in the slurry density as a function of dilutions, is expected for insoluble solids. The decrease in the centrifuged supernate density as a function of dilution also indicates insoluble solids. These conclusions do not exclude the possibility that some components of the solids are soluble, but these soluble components are not the major components of solids. The weight percent dissolved solids indicate that a significant amount of salts are dissolved in the centrifuged supernate of the as-received sample, but no analysis was performed on the dilutions to determine the amount of solids dissolved during each dilution.

The 1:1 dilution for core 50, segment 2, reached a final volume percent settled solids behavior of 65 to 75 percent. Settling was observed throughout the two-day period, but the majority of the settling was observed in the first 10 hours. The 3:1 dilution reached a final volume percent settled solids of approximately 32 percent. Significant settling for both dilutions was observed over the 48 hours, but the settling velocity of these dilutions decreased sharply over the first eight hours, then remained constant. After the drop in

percent settled solids, the remainder of the suspended solids, approximately 25 percent, took up most of the remaining time settling in a long, gradual decline, before coming to equilibrium.

Table B2-6. Physical Properties Summary (Core 50, Segment 2).

Property	As Received	1:1	3:1
Settled solids (vol%)	100	68	32
Centrifuged solids (vol%)	74	36	16
Centrifuged solids (wt%)	79	44	19
Total solids (wt%)	47	NA	NA
Dissolved solids (wt%)	22	NA	NA
Undissolved solids (wt%)	25	NA	NA
Density (g/mL)	As Received	1:1	3:1
Sample	1.44 ¹	1.22	1.10
Centrifuged supernate	1.20	1.07	1.03
Centrifuged solids	1.53	1.44	1.32

Notes:

NA = Not analyzed

¹Density results were obtained from the 325 Laboratory.

The weight percent water was determined on all segments for both cores and the core composite samples. Two methods were used: gravimetry and thermogravimetry. The gravimetric determination of the weight percent water is measured by the loss of mass in the sample after being held in a drying oven at 102 °C (216 °F) for 24 hours. Table B2-90 shows the results. In addition, the weight percent residual solids and the mass of the residual solids were measured on the core composites after water digestion. Tables B2-91 and B2-92 show the results. Table B2-93 shows the total dissolved solids as measured on the solids composites.

Rheological Properties. The shear strength of the waste from tank 241-T-107 was measured on unhomogenized samples. Because only one visually discernible stratum was observed in tank 241-T-107, shear strength measurements were only performed on core 50, segment 2. The shear strength measurements were made at ambient temperature using a shear vane connected to a viscometer and rotated at 0.3 rpm.

Shear strength (τ_s) is a semiquantitative measurement of the force required to move the sample. The shear strength of the sample was measured at four different locations. The average shear strength was 7,200 dynes/cm² (see Table B2-94); the standard deviation for these four measurements was 3,700 dynes/cm². This large variance between measurements is attributed largely to the heterogeneity of the sample. The heterogeneity of this sample is evident in the moisture content. There is an observable relationship to the moisture content; core 50, segment 2, has 29.8 percent water for an unhomogenized sample (43 percent for homogenized sample). The percent water does not compare with adjoining segments which have a significantly higher amount of water present. The torque on the sample was recorded as a function of time and the shear strength was calculated using the following equation.

$$\tau_s = \frac{[\% \tau / 100] * S_r * 4.9E+05}{\frac{\pi * H_v * D_v^2}{2} + \frac{\pi * D_v^3}{6}}$$

where:

$\% \tau / 100$	=	the ratio of the total torque to the maximum torque of the viscometer head, measured as a percentage of the full scale on the plot of the shear stress versus time diagram (dimensionless)
S_r	=	signal (reading) proportional to the torque
4.9E+05	=	maximum torque of the viscometer head (dynes•cm)
H_v	=	shear vane height (1.582 cm)
D_v	=	shear vane diameter (0.800 cm)

Although relatively low, the shear strength of the material substantially exceeded the baseline value for the measurement system (200 dynes/cm²).

Shear Stress and Viscosity. Shear stress measurements, as functions of shear rate, were performed on the 1:1 and 3:1 (water:sample) dilution of the sample at ambient hot cell temperatures. Because of drying of the sample on the plate at elevated temperatures, the shear stress of the samples as a function of shear rate could not be measured on the as-received samples at 95 °C.

A rheogram for a material with a yield has two sections. The first section is a straight line beginning at the origin and climbing up the ordinate. This portion of the rheogram records the material as it acts like a solid or gel. When sufficient force is applied to the material to make the gel yield, the rheogram breaks sharply to the right, recording the material's behavior as a fluid. The point on the rheogram at which the sample's behavior transfers from a solid or gel to a fluid is the yield point or yield stress. The minimum shear stress must be exceeded to initiate fluid behavior in the material. The samples are elastic under low shear conditions (less than 50 s^{-1}) and plastic under high shear conditions (greater than 300 s^{-1}). The general behavior exhibited by the waste is best described by a yield pseudoplastic model; however, the systems were not modeled, and empirical model parameters were not determined because the system was at the limits of detection.

The 1:1 dilution samples have significant yield points at approximately 0.75 Pa; therefore, the 1:1 dilution samples exhibit yield pseudoplastic behavior. The 3:1 dilutions exhibit essentially Newtonian behavior. Table B2-95 shows the yield point, consistency factor, and flow behavior index data for the 1:1 dilution samples.

The viscosity of this sample (core 50, segment 2) ranges from 20 to 9 centipoise (cP) over a shear rate range of approximately 100 to 400 s^{-1} . The viscosity of the sample decreases with increasing shear rate. At $90 \text{ }^\circ\text{C}$, the viscosity of the sample was slightly lower (12 to 7 cP over a shear rate range of 100 to 400 s^{-1}) than at ambient temperature. At shear rates greater than 100 s^{-1} , the viscosity of the 3:1 dilution was less than or equal to 5 cP. At $95 \text{ }^\circ\text{C}$, the viscosity of the 3:1 dilution is lower than was observed at ambient temperature (less than or equal to 3 cP at shear rates greater than 100 s^{-1}); thus, it appears that the viscosity of the samples decreases with increasing temperature.

Particle Size Analysis. To evaluate which potential waste retrieval method will be done for each tank, a particle size analysis was performed according to procedure T044-A-01712F. Particle size analysis was performed once on tank 241-T-107 because, at the time of extrusion and sample breakdown, the hot cell chemist visually observed one stratum only.

An important consideration involving the analysis of particle size is the dispersant (the liquid used to disperse and suspend the particles from the solid sample) used. The primary concern with the dispersant is dissolving the particles present. Any particles in the tank that are soluble in the dispersant will dissolve or decrease in size during the analysis. Depending on the dispersant, the particle size analysis may not represent the true particle size distribution in the tank. In the case of tank 241-241-T-107, water was used as the dispersing medium. If a "true" particle size distribution is required, the mother liquor (drainable liquid) of the tank should be used, if possible, because the tank particulates are already in equilibrium with the tank mother liquor.

To perform particle size analysis, a small aliquot of waste is placed in a dispersant (water) to separate and suspend particles. The waste/water matrix is placed in the particle size analyzer, and a beam of laser light is passed through the dispersant. The diameter of solid particles can be determined by the amount of light that passes through the matrix. There are

two distinct ways the analyzer determines particle size: number distribution range and volume distribution range.

Tank 241-T-107 had only one particle size analysis, performed on core 50, segment 2. The analysis was performed on the unhomogenized sample. The particle sizes for core 50, segment 2, are as follows: number distribution range = 0.5 to 8 μm with a mean of 1.09 μm , and a majority of the volume distribution range = 0.10 to 150 μm with a mean of 39.05 μm . Some particles may have been greater than 150 μm , but this number was the upper limit on the analyzer. Refer to Figures 5-8 and 5-9 in Valenzuela and Jensen (1994) for a graphical representation. Table B2-7 summarizes the particle size results.

Table B2-7. Particle Size Data for Tank 241-T-107.

Core 50, Segment 2	Distribution Range	Mean	Median
Number distribution	0.5 to 8.0 μm	1.09 μm	0.85 μm
Volume distribution	0.10 to 150 μm	39.05 μm	32.97 μm

B2.1.4.6 Thermodynamic Analyses. The following subsections discuss the thermodynamic analyses performed on the tank 241-T-107 waste. These analyses include DSC and TGA.

Differential Scanning Calorimetry. In a DSC analysis, heat absorbed or emitted by a substance is measured while the temperature of the sample is heated at a constant rate. Nitrogen is passed over the sample material to remove any gases being released. The onset temperature for an endothermic or exothermic event is determined graphically.

The DSC analyses for tank 241-T-107 were performed using either procedure LA-514-113 on a Mettler³ DSC 20 instrument or procedure LA-514-114 on a Perkin-Elmer⁴ DSC 7 instrument. The DSC traces for segment 4 show an exotherm beginning around 300 °C. This exothermic region was attributed to a plastic artifact that was commingled with the waste in the last segment of core 50. Further analysis of the plastic artifact with different carrier gases showed the artifact was anomalous and did not represent tank waste (Svancara and Pool 1993). No exothermic reactions were noted that were attributed to the waste; therefore, DSC results are not shown in the analytical tables, and 95 percent confidence intervals on the mean for each sample were not calculated.

³Mettler is a registered trademark of Mettler Electronics, Anaheim, CA.

⁴Perkin Elmer is a registered trademark of Perkins Research and Manufacturing Company, Inc., Canoga Park, CA.

Thermogravimetric Analysis. Thermogravimetric analysis measures the mass of a sample while its temperature is increased at a constant rate. Air is passed over the sample during heating to remove any released gases. Any decrease in the weight of a sample during TGA represents a loss of gaseous matter from the sample, through evaporation or through a reaction that forms gas phase products. The moisture content is estimated by assuming that all TGA sample weight loss up to a certain temperature (typically 150 to 200 °C) is caused by water evaporation. The temperature limit for moisture loss is chosen by the operator at an inflection point on the TGA plot. Other volatile matter fractions can be differentiated by inflection points as well.

Tank 241-T-107 samples were analyzed by TGA using procedure LA-560-112 on a Mettler® TG 50 instrument. Table B2-96 shows TGA results. Gravimetric analyses were performed on tank subsamples to assess the accuracy of the TGA results. Because some TGA results were interpreted conservatively, the gravimetric results provide a better estimate of sample moisture content.

The TGA was performed on nonhomogenized facies, homogenized segments or subsegments, and drainable liquid composites. The values produced may vary substantially as a result of the small sample size and sample heterogeneity. In core 50, segment 4, an anomalous percent water was noted which was attributed to the plastic material that was burned with air cover gas; therefore, the TGA was not measuring the water content of this sample. When the cover gas was changed to nitrogen, no loss in weight was noted.

Gravimetrically measuring the amount of solids provides more representative measurements of the water/solids content within a sample. The gravimetric method uses a larger sample aliquot than the TGA (about 1 g versus 10 to 35 mg), reducing variations caused by sample heterogeneity. The samples are heated in an oven at 102 °C (216 °F) until the weight measurements do not change, indicating all free water has been removed. All solids composites and homogenized segments or subsegments (except core 50, segments 3 and 4, and core 52, segment 4) were analyzed in duplicate by this method according to procedure LA-564-101. See Section B3.3.1 for a comparison of the TGA and gravimetric data.

B2.1.4.7 Data Tables. The data tables in this section were footnoted when standard recoveries, spike recoveries, or duplicate analyses were outside the QC criteria specified in Bell (1993). The QC criteria specified for sample analysis for ICP is as follows: 75 to 125 percent recovery for spikes, 80 to 120 percent recovery for standards, and ± 20 percent relative percent differences (RPDs) for duplicates; for the remaining analytes, 80 to 120 percent recovery for both spikes and standards and ± 20 percent for RPDs. Sample and duplicate pairs, in which these QC parameters were outside of these limits, are footnoted in the sample mean column of the data summary tables as follows:

- "a" indicates the standard recovery was below the QC limit.
- "b" indicates the standard recovery was above the QC limit.

- "c" indicates the spike recovery was below the QC limit.
- "d" indicates the spike recovery was above the QC limit.
- "e" indicates the RPD was above the QC limit.
- "f" indicates the blank was contaminated.

A complete data validation was performed on the data set as described in Svancara and Pool (1993). Any data qualified as unusable have been appropriately footnoted in the data tables as follows:

- "r" indicates the data was qualified as unusable.

See Section B3.2 for a discussion of the data validation results.

Table B2-8. Tank 241-T-107 Analytical Results: Cesium (AA).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion			$\mu\text{g/g}$	$\mu\text{g/g}$	$\mu\text{g/g}$
436-6793	Core 51 composite	Whole	< 700	< 700	< 700
500-6793	Core 52 composite	Whole	< 700	< 700	< 700
Solids: acid digest			$\mu\text{g/g}$	$\mu\text{g/g}$	$\mu\text{g/g}$
439-8794	Core 51 composite	Whole	< 140	< 140	< 140

Table B2-9. Tank 241-T-107 Analytical Results: Mercury (CVAA).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids			$\mu\text{g/g}$	$\mu\text{g/g}$	$\mu\text{g/g}$
427-5798	Core 51 composite	Whole	<0.125	<0.125	<0.125
495-5798	Core 52 composite	Whole	0.153	0.134	0.1435

Table B2-10. Tank 241-T-107 Analytical Results: Aluminum (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: acid digest			µg/g	µg/g	µg/g
356	50: 2	Whole	87,100	86,900	87,000 ^{QC:b}
356		Whole	1.320E+05	79,000	1.055E+05 ^{QC:b,e}
357		Whole	92,900	68,100	80,500 ^{QC:b,e}
357		Whole	95,800	1.220E+05	1.089E+05 ^{QC:b,e}
418	51: 3	Lower half	915	---	915 ^{QC:b}
418		Lower half	254	83.9	168.95 ^{QC:b,e,r}
419		Lower half	206	235	220.5 ^{QC:b,r}
419		Lower half	438	385	411.5 ^{QC:b}
490	52: 3	Lower half	16,000	16,400	16,200
490		Lower half	15,800	15,300	15,550 ^{QC:b}
491		Lower half	18,500	18,700	18,600
491		Lower half	15,900	15,300	15,600 ^{QC:b}
549	Core 51 composite	Whole	4,030	4,250	4,140
503	Core 52 composite	Whole	23,900	25,300	24,600 ^{QC:b,d}
Solids: fusion			µg/g	µg/g	µg/g
387	50: 1R	Subsegment	9,900	9,720	9,810
309	50: 2	Whole	90,900	94,900	92,900 ^{QC:d}
322	50: 3	Whole	18,500	23,000	20,750 ^{QC:e}
406	51: 2	Whole	11,300	12,700	12,000
255	51: 3	Lower half	586	790	688 ^{QC:e}
254		Upper half	1,340	1,150	1,245
257	51: 4	Lower half	9,270	9,160	9,215
256		Upper half	2,220	2,320	2,270
455	52: 1	Whole	2.120E+05	2.150E+05	2.135E+05 ^{QC:d}
481	52: 2	Whole	46,300	39,800	43,050
483	52: 3	Lower half	15,200	15,600	15,400
482		Upper half	7,810	8,530	8,170

Table B2-10. Tank 241-T-107 Analytical Results: Aluminum (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion (Cont'd)			µg/g	µg/g	µg/g
436	Core 51 composite	Whole	7,260	4,200	5,730 ^{QC:e}
495	Core 52 composite	Whole	26,700	27,800	27,250
500		Whole	26,400	27,500	26,950
Solids: water digest			µg/g	µg/g	µg/g
534	Core 51 composite	Whole	344	627	485.5 ^{QC:e}
495	Core 52 composite	Whole	849	784	816
Liquids			µg/mL	µg/mL	µg/mL
393	Core 50 composite	Drainable liquid	4.45	4.06	4.255 ^{QC:b}
442	Core 51 composite	Drainable liquid	11.1	11.8	11.45 ^{QC:b}
494	Core 52 composite	Drainable liquid	47.2	48.4	47.8 ^{QC:b}

Table B2-11. Tank 241-T-107 Analytical Results: Antimony (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: acid digest			µg/g	µg/g	µg/g
356	50: 2	Whole	134	87.4	110.7 ^{QC:c,e}
356		Whole	84.3	60.2	72.25 ^{QC:b,c}
357		Whole	123	113	118
357		Whole	80.4	68.4	74.4 ^{QC:b,d}
418	51: 3	Lower half	273	---	273
418		Lower half	153	138	145.5 ^{QC:a,r}
419		Lower half	228	220	224 ^{QC:c}
419		Lower half	158	151	154.5 ^{QC:a,r}
490	52: 3	Lower half	< 20.8	< 20.8	< 20.8 ^{QC:a}
490		Lower half	29.5	< 20.7	< 25.1 ^{QC:a,c,e}
491		Lower half	20.7	< 20.9	< 20.8 ^{QC:a}
491		Lower half	24.3	< 20.8	< 22.55 ^{QC:a,c}
549	Core 51 composite	Whole	< 209	< 208	< 208.5
503	Core 52 composite	Whole	37.4	53.6	45.5 ^{QC:d,e}
Solids: fusion			µg/g	µg/g	µg/g
387	50: 1R	Subsegment	< 75.2	< 75.1	< 75.15
309	50: 2	Whole	78.3	190	134.15 ^{QC:c}
322	50: 3	Whole	128	118	123
406	51: 2	Whole	108	77.4	92.7 ^{QC:c}
255	51: 3	Lower half	230	160	195 ^{QC:c}
254		Upper half	115	110	112.5
257	51: 4	Lower half	< 73.4	94.2	< 83.8
256		Upper half	181	183	182
455	52: 1	Whole	< 1,030	< 1,030	< 1,030
481	52: 2	Whole	82.8	< 75.7	< 79.25
483	52: 3	Lower half	142	< 75	< 108.5 ^{QC:c}
482		Upper half	113	127	120

Table B2-11. Tank 241-T-107 Analytical Results: Antimony (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: acid digest			µg/g	µg/g	µg/g
356	50: 2	Whole	134	87.4	110.7 ^{QC:c,e}
Solids: fusion (Cont'd)			µg/g	µg/g	µg/g
436	Core 51 composite	Whole	139	104	121.5 ^{QC:d,e}
495	Core 52 composite	Whole	<1,040	<1,040	<1,040
500		Whole	<75.4	<75.8	<75.6
Solids: water digest			µg/g	µg/g	µg/g
534	Core 51 composite	Whole	<22.1	<22.1	<22.1
495	Core 52 composite	Whole	<213	<213	<213
Liquids			µg/mL	µg/mL	µg/mL
393	Core 50 composite	Drainable liquid	1.57	2.27	1.92 ^{QC:c}
442	Core 51 composite	Drainable liquid	2.52	2.52	2.52
494	Core 52 composite	Drainable liquid	<3.8	<3.8	<3.8 ^{QC:b}

Table B2-12. Tank 241-T-107 Analytical Results: Arsenic (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: acid digest			µg/g	µg/g	µg/g
356	50: 2	Whole	6.27	6.66	6.465
356		Whole	<2.92	<3.00	<2.96
357		Whole	<2.96	<2.88	<2.92
357		Whole	6.86	<3.9	<5.38 ^{QC:c}
418	51: 3	Lower half	<2.3	---	<2.3
418		Lower half	4.53	4.02	4.275
419		Lower half	4.71	4.65	4.68
419		Lower half	<2.99	<2.95	<2.97
490	52: 3	Lower half	<3	<2.95	<2.975
490		Lower half	<2.98	4.5	<3.74 ^{QC:c}
491		Lower half	<2.95	<2.99	<2.97
491		Lower half	<2.96	<2.96	<2.96
549	Core 51 composite	Whole	<29.9	<29.8	<29.85
503	Core 52 composite	Whole	4.63	3.95	4.29
Solids: fusion			µg/g	µg/g	µg/g
387	50: 1R	Subsegment	<19.3	<19.3	<19.3
309	50: 2	Whole	22.3	<19.4	<20.85
322	50: 3	Whole	<19.3	<19.3	<19.3
406	51: 2	Whole	<19.4	<19.4	<19.4
255	51: 3	Lower half	<19	<19.5	<19.25
254		Upper half	<18.3	<18	<18.15
257	51: 4	Lower half	<18.8	<18.7	<18.75
256		Upper half	<18.8	<19	<18.9
455	52: 1	Whole	<148	<147	<147.5
481	52: 2	Whole	23.5	<19.4	<21.45
483	52: 3	Lower half	<19.4	<19.2	<19.3
482		Upper half	<19.3	<19.2	<19.25

Table B2-12. Tank 241-T-107 Analytical Results: Arsenic (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion (Cont'd)			µg/g	µg/g	µg/g
436	Core 51 composite	Whole	< 19.4	<19.3	<19.35
495	Core 52 composite	Whole	<148	<148	<148
500		Whole	<19.3	<19.5	<19.4
Solids: water digest			µg/g	µg/g	µg/g
534	Core 51 composite	Whole	<3.16	<3.16	<3.16
495	Core 52 composite	Whole	<30.4	<30.4	<30.4
Liquids			µg/mL	µg/mL	µg/mL
393	Core 50 composite	Drainable liquid	<0.39	<0.39	<0.39
442	Core 51 composite	Drainable liquid	<0.39	<0.39	<0.39
494	Core 52 composite	Drainable liquid	<0.975	<0.975	<0.975

Table B2-13. Tank 241-T-107 Analytical Results: Beryllium (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: acid digest			$\mu\text{g/g}$	$\mu\text{g/g}$	$\mu\text{g/g}$
356	50: 2	Whole	<0.299	<0.299	<0.299
356		Whole	<0.292	<0.3	<0.296 ^{QC:c}
357		Whole	<0.299	<0.3	<0.2995
357		Whole	<0.296	<0.288	<0.292
418	51: 3	Lower half	<0.23	---	<0.23
418		Lower half	<0.299	<0.297	<0.298
419		Lower half	<0.299	<0.296	<0.2975
419		Lower half	<0.299	<0.295	<0.297
490	52: 3	Lower half	<0.3	<0.295	<0.2975
490		Lower half	<0.298	<0.296	<0.297
491		Lower half	0.314	0.346	0.33
491		Lower half	<0.295	<0.299	<0.297
549	Core 51 composite	Whole	<2.99	<2.98	<2.985
503	Core 52 composite	Whole	<0.299	<0.297	<0.298
Solids: fusion			$\mu\text{g/g}$	$\mu\text{g/g}$	$\mu\text{g/g}$
387	50: 1R	Subsegment	<1.49	<1.48	<1.485
309	50: 2	Whole	<1.5	<1.49	<1.495
322	50: 3	Whole	<1.49	<1.49	<1.49
406	51: 2	Whole	<1.49	<1.49	<1.49
255	51: 3	Lower half	<1.46	<1.5	<1.48
254		Upper half	<1.41	<1.39	<1.4
257	51: 4	Lower half	<1.45	<1.44	<1.445
256		Upper half	<1.44	<1.46	<1.45
455	52: 1	Whole	<14.8	<14.7	<14.75
481	52: 2	Whole	<1.49	<1.49	<1.49
483	52: 3	Lower half	<1.49	<1.48	<1.485
482		Upper half	<1.49	<1.48	<1.485

Table B2-13. Tank 241-T-107 Analytical Results: Beryllium (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion (Cont'd)			µg/g	µg/g	µg/g
436	Core 51 composite	Whole	<1.49	<1.49	<1.49
495	Core 52 composite	Whole	<14.8	<14.8	<14.8
500		Whole	<1.49	<1.5	<1.495
Solids: water digest			µg/g	µg/g	µg/g
534	Core 51 composite	Whole	<0.316	<0.316	<0.316
494	Core 52 composite	Whole	<3.04	<3.04	<3.04
Liquids			µg/mL	µg/mL	µg/mL
393	Core 50 composite	Drainable liquid	<0.03	<0.03	<0.03
442	Core 51 composite	Drainable liquid	<0.03	<0.03	<0.03
494	Core 52 composite	Drainable liquid	<0.075	<0.075	<0.075

Table B2-14. Tank 241-T-107 Analytical Results: Bismuth (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: acid digest			µg/g	µg/g	µg/g
356	50: 2	Whole	3,110	2,530	2,820 ^{QC:a,c}
356		Whole	4,030	2,800	3,415 ^{QC:a,d,e}
357		Whole	2,950	3,100	3,025 ^{QC:a,c}
357		Whole	3,870	3,610	3,740 ^{QC:a,d}
418	51: 3	Lower half	1,920	---	1,920 ^{QC:a}
418		Lower half	1,050	1,110	1,080 ^{QC:a,d}
419		Lower half	1,160	1,050	1,105 ^{QC:a}
419		Lower half	1,530	1,390	1,460 ^{QC:a,c}
490	52: 3	Lower half	23,700	24,900	24,300 ^{QC:a}
490		Lower half	24,600	24,200	24,400 ^{QC:a}
491		Lower half	24,700	23,900	24,300 ^{QC:a}
491		Lower half	27,800	28,000	27,900 ^{QC:a}
549	Core 51 composite	Whole	7,790	8,130	7,960 ^{QC:a}
503	Core 52 composite	Whole	13,100	14,400	13,750 ^{QC:a}
Solids: fusion			µg/g	µg/g	µg/g
387	50: 1R	Subsegment	20,500	18,300	19,400 ^{QC:c}
309	50: 2	Whole	2,710	2,520	2,615 ^{QC:c}
322	50: 3	Whole	10,600	11,400	11,000 ^{QC:c}
406	51: 2	Whole	986	987	986.5
255	51: 3	Lower half	1,090	1,040	1,065
254		Upper half	1,000	866	933 ^{QC:c}
257	51: 4	Lower half	19,600	17,500	18,550
256		Upper half	3,510	3,730	3,620
455	52: 1	Whole	<310	376	<343
481	52: 2	Whole	11,500	10,700	11,100
483	52: 3	Lower half	22,100	22,600	22,350 ^{QC:c}
482		Upper half	9,580	10,900	10,240 ^{QC:c}

Table B2-14. Tank 241-T-107 Analytical Results: Bismuth (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion (Cont'd)			µg/g	µg/g	µg/g
436	Core 51 composite	Whole	8,200	8,490	8,345 ^{QC,c}
495	Core 52 composite	Whole	12,900	12,300	12,600
500		Whole	14,000	17,300	15,650 ^{QC:c,e}
Solids: water digest			µg/g	µg/g	µg/g
534	Core 51 composite	Whole	108	81.5	94.75 ^{QC:c,e}
495	Core 52 composite	Whole	375	409	392
Liquids			µg/mL	µg/mL	µg/mL
393	Core 50 composite	Drainable liquid	<0.92	<0.92	<0.92 ^{QC:a}
442	Core 51 composite	Drainable liquid	<0.92	<0.92	<0.92 ^{QC:a,c}
494	Core 52 composite	Drainable liquid	16.9	17	16.95 ^{QC:a}

Table B2-15. Tank 241-T-107 Analytical Results: Boron (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: acid digest			µg/g	µg/g	µg/g
356	50: 2	Whole	6.7	5.91	6.305 ^{QC:b,c}
356		Whole	<0.998	6.57	<3.784 ^{QC:b,e}
357		Whole	6.46	11.9	9.18 ^{QC:b,c,e}
357		Whole	<0.998	1.55	<1.274 ^{QC:b,e}
418	51: 3	Lower half	43.6	---	43.6 ^{QC:b}
418		Lower half	<1.74	<0.99	<1.365 ^{QC:b,e}
419		Lower half	17.8	14.1	15.95 ^{QC:b,e}
419		Lower half	<0.997	<0.988	<0.9925 ^{QC:b}
490	52: 3	Lower half	18.3	15.1	16.7 ^{QC:b}
490		Lower half	25.2	19.8	22.5 ^{QC:b,e}
491		Lower half	19	15.8	17.4 ^{QC:b}
491		Lower half	23.6	14.3	18.95 ^{QC:b,e}
549	Core 51 composite	Whole	33.8	13.5	23.65 ^{QC:b,e}
503	Core 52 composite	Whole	3.59	<0.99	<2.29 ^{QC:b,e}
Solids: fusion			µg/g	µg/g	µg/g
387	50: 1R	Subsegment	6.98	6.92	6.95
309	50: 2	Whole	<5	<4.97	<4.985
322	50: 3	Whole	<4.95	<4.95	<4.95
406	51: 2	Whole	<4.98	<4.98	<4.98
255	51: 3	Lower half	<4.87	<5	<4.935
254		Upper half	<4.7	<4.62	<4.66
257	51: 4	Lower half	8.64	<4.8	<6.72 ^{QC:e}
256		Upper half	<4.81	<4.86	<4.835
455	52: 1	Whole	<49.2	<49.1	<49.15 ^{QC:d}
481	52: 2	Whole	<4.98	<4.98	<4.98
483	52: 3	Lower half	8.02	11.8	9.91 ^{QC:e}
482		Upper half	<4.96	<4.92	<4.94

Table B2-15. Tank 241-T-107 Analytical Results: Boron (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion (Cont'd)			µg/g	µg/g	µg/g
436	Core 51 composite	Whole	<4.98	<4.95	<4.965
495	Core 52 composite	Whole	<49.3	<49.4	<49.35 ^{QC,d}
500		Whole	<4.96	<4.99	<4.975
Solids: water digest			µg/g	µg/g	µg/g
534	Core 51 composite	Whole	607	666	636.5
495	Core 52 composite	Whole	16.1	14.8	15.5
Liquids			µg/mL	µg/mL	µg/mL
393	Core 50 composite	Drainable liquid	9.33	9.03	9.18 ^{QC,b,d}
442	Core 51 composite	Drainable liquid	24.2	25	24.6 ^{QC,b,d}
494	Core 52 composite	Drainable liquid	28.3	32	30.15 ^{QC,b,d,r}

Table B2-16. Tank 241-T-107 Analytical Results: Cadmium (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: acid digest			µg/g	µg/g	µg/g
356	50: 2	Whole	5.6	4.65	5.125
356		Whole	7.52	5.21	6.365 ^{QC:c,e}
357		Whole	5.5	5.43	5.465
357		Whole	6.94	6.84	6.89
418	51: 3	Lower half	12.3	---	12.3
418		Lower half	5.77	5.72	5.745
419		Lower half	5.91	5.66	5.785
419		Lower half	7.47	6.62	7.045
490	52: 3	Lower half	1.41	1.3	1.355
490		Lower half	3.99	3.1	3.545 ^{QC:a}
491		Lower half	1.55	1.05	1.3 ^{QC:c,e}
491		Lower half	3.94	4.07	4.005 ^{QC:a}
549	Core 51 composite	Whole	7.93	6.5	7.215 ^{QC:a}
503	Core 52 composite	Whole	4.17	7.01	5.59 ^{QC:c,e}
Solids: fusion			µg/g	µg/g	µg/g
387	50: 1R	Subsegment	4.68	6.83	5.755 ^{QC:c,e}
309	50: 2	Whole	6.58	6.41	6.495
322	50: 3	Whole	6.65	6.49	6.57
406	51: 2	Whole	40.6	16.7	28.65 ^{QC:c,e}
255	51: 3	Lower half	7.52	6.53	7.025
254		Upper half	7.84	7.6	7.72
257	51: 4	Lower half	5.57	7.51	6.54 ^{QC:c,e}
256		Upper half	8.27	10.4	9.335 ^{QC:c,e}
455	52: 1	Whole	< 29.5	< 29.5	< 29.5
481	52: 2	Whole	4.59	5.19	4.89
483	52: 3	Lower half	5.92	6.08	6
482		Upper half	6.22	4.78	5.5 ^{QC:c,e}

Table B2-16. Tank 241-T-107 Analytical Results: Cadmium (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion (Cont'd)			µg/g	µg/g	µg/g
50: 1R	Subsegment	4.68	6.83	5.755 ^{QC:e}	6.15
495	Core 52 composite	Whole	<29.6	<29.6	<29.6
500		Whole	5.66	9.81	7.735 ^{QC:e}
Solids: water digest			µg/g	µg/g	µg/g
534	Core 51 composite	Whole	<0.632	<0.632	<0.632
495	Core 52 composite	Whole	<6.07	<6.07	<6.07
Liquids			µg/mL	µg/mL	µg/mL
393	Core 50 composite	Drainable liquid	<0.07	<0.07	<0.07
442	Core 51 composite	Drainable liquid	<0.07	<0.07	<0.07
494	Core 52 composite	Drainable liquid	<0.175	<0.175	<0.175

Table B2-17. Tank 241-T-107 Analytical Results: Calcium (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: acid digest			µg/g	µg/g	µg/g
356	50: 2	Whole	658	516	587 ^{QC:b,e}
356		Whole	1,520	1,470	1,495 ^{QC:b,c}
357		Whole	620	643	631.5 ^{QC:b,d}
357		Whole	4,690	942	2,816 ^{QC:b,c,e}
418	51: 3	Lower half	671	723	697 ^{QC:d}
418		Lower half	1,090	---	1,090 ^{QC:b}
419		Lower half	735	672	703.5
419		Lower half	906	843	874.5 ^{QC:b,c}
490	52: 3	Lower half	449	449	449 ^{QC:b}
490		Lower half	375	405	390 ^{QC:b,d}
491		Lower half	420	409	414.5 ^{QC:b}
491		Lower half	883	603	743 ^{QC:b,c,e}
549	Core 51 composite	Whole	808	897	852.5 ^{QC:b}
503	Core 52 composite	Whole	542	643	592.5 ^{QC:b}
Solids: fusion			µg/g	µg/g	µg/g
387	50: 1R	Subsegment	1,060	1,040	1,050
309	50: 2	Whole	895	749	822
322	50: 3	Whole	1,100	910	1,005
406	51: 2	Whole	2,160	2,030	2,095
255	51: 3	Lower half	956	1,020	988
254		Upper half	963	959	961
257	51: 4	Lower half	2,060	2,810	2,435 ^{QC:e}
256		Upper half	1,400	1,460	1,430
455	52: 1	Whole	13,500	8,300	10,900 ^{QC:e}
481	52: 2	Whole	671	871	771
483	52: 3	Lower half	437	407	422
482		Upper half	583	1,020	801.5 ^{QC:e}

Table B2-17. Tank 241-T-107 Analytical Results: Calcium (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion (Cont'd)			µg/g	µg/g	µg/g
436	Core 51 composite	Whole	765	794	779.5
495	Core 52 composite	Whole	6,780	611	3,695.5 ^{QC:c}
500		Whole	702	781	741.5
Solids: water digest			µg/g	µg/g	µg/g
534	Core 51 composite	Whole	151	800	475.5 ^{QC:c}
495	Core 52 composite	Whole	71.1	59.9	65.5
Liquids			µg/mL	µg/mL	µg/mL
393	Core 50 composite	Drainable liquid	4.87	3.7	4.285 ^{QC:c}
442	Core 51 composite	Drainable liquid	7	2.95	4.975 ^{QC:c}
494	Core 52 composite	Drainable liquid	3.48	4.4	3.94 ^{QC:b,r}

Table B2-18. Tank 241-T-107 Analytical Results: Cerium (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: acid digest			µg/g	µg/g	µg/g
356	50: 2	Whole	27.4	20.6	24 ^{QC:c}
356		Whole	< 6.61	< 6.8	< 6.705 ^{QC:c}
357		Whole	36.8	24.1	30.45 ^{QC:c}
357		Whole	80.4	297	188.7 ^{QC:c,e}
418	51: 3	Lower half	< 5.3	---	< 5.3 ^{QC:c,r}
418		Lower half	< 12.8	< 12.7	< 12.75
419		Lower half	< 12.8	< 12.6	< 12.7
419		Lower half	< 6.77	< 6.69	< 6.73 ^{QC:c,r}
490	52: 3	Lower half	266	282	274
490		Lower half	327	314	320.5
491		Lower half	295	295	295
491		Lower half	328	318	323
549	Core 51 composite	Whole	< 67.8	< 67.5	< 67.65 ^{QC:a}
503	Core 52 composite	Whole	180	185	182.5
Solids: fusion			µg/g	µg/g	µg/g
387	50: 1R	Subsegment	133	113	123
309	50: 2	Whole	< 64	< 63.6	< 63.8
322	50: 3	Whole	< 63.4	78.7	< 71.05
406	51: 2	Whole	< 63.7	< 63.7	< 63.7
255	51: 3	Lower half	< 62.4	< 64	< 63.2
254		Upper half	< 60.2	< 59.1	< 59.65
257	51: 4	Lower half	212	226	219
256		Upper half	67.9	< 62.3	< 65.1
455	52: 1	Whole	< 335	< 334	< 334.5
481	52: 2	Whole	202	158	180 ^{QC:c}
483	52: 3	Lower half	376	308	342
482		Upper half	207	235	221

Table B2-18. Tank 241-T-107 Analytical Results: Cerium (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion (Cont'd)			µg/g	µg/g	µg/g
436	Core 51 composite	Whole	87.6	104	95.8
495	Core 52 composite	Whole	<335	<336	<335.5
500		Whole	132	137	134.5
Solids: water digest			µg/g	µg/g	µg/g
534	Core 51 composite	Whole	<7.16	<7.16	<7.16
495	Core 52 composite	Whole	<68.8	<68.9	<68.9
Liquids			µg/mL	µg/mL	µg/mL
393	Core 50 composite	Drainable liquid	<1.28	<1.28	<1.28
442	Core 51 composite	Drainable liquid	<1.28	<1.28	<1.28
494	Core 52 composite	Drainable liquid	<3.2	<3.2	<3.2

Table B2-19. Tank 241-T-107 Analytical Results: Chromium (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: acid digest			µg/g	µg/g	µg/g
356	50: 2	Whole	279	229	254
356		Whole	329	233	281 ^{QC:d,e}
357		Whole	278	286	282 ^{QC:c}
357		Whole	314	289	301.5 ^{QC:d}
418	51: 3	Lower half	494	---	494
418		Lower half	336	354	345 ^{QC:d}
419		Lower half	355	334	344.5
419		Lower half	415	387	401 ^{QC:c}
490	52: 3	Lower half	532	563	547.5 ^{QC:d}
490		Lower half	593	572	582.5 ^{QC:c}
491		Lower half	593	561	577 ^{QC:c}
491		Lower half	623	608	615.5 ^{QC:c}
549	Core 51 composite	Whole	383	381	382
503	Core 52 composite	Whole	309	342	325.5
Solids: fusion			µg/g	µg/g	µg/g
387	50: 1R	Subsegment	339	327	333
309	50: 2	Whole	264	252	258
322	50: 3	Whole	205	233	219
406	51: 2	Whole	370	341	355.5
255	51: 3	Lower half	392	395	393.5
254		Upper half	321	280	300.5
257	51: 4	Lower half	355	342	348.5
256		Upper half	480	504	492
455	52: 1	Whole	123	161	142 ^{QC:c}
481	52: 2	Whole	256	242	249
483	52: 3	Lower half	545	516	530.5
482		Upper half	275	296	285.5

Table B2-19. Tank 241-T-107 Analytical Results: Chromium (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion (Cont'd)			µg/g	µg/g	µg/g
436	Core 51 composite	Whole	351	359	355
495	Core 52 composite	Whole	280	348	314 ^{QC:c}
500		Whole	341	389	365
Solids: water digest			µg/g	µg/g	µg/g
534	Core 51 composite	Whole	230	216	223
495	Core 52 composite	Whole	213	184	198
Liquids			µg/mL	µg/mL	µg/mL
393	Core 50 composite	Drainable liquid	45.6	45	45.3
442	Core 51 composite	Drainable liquid	271	275	273 ^{QC:d}
494	Core 52 composite	Drainable liquid	186	190	188

Table B2-20. Tank 241-T-107 Analytical Results: Iron (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: acid digest			µg/g	µg/g	µg/g
356	50: 2	Whole	21,600	20,100	20,850
356		Whole	31,800	21,000	26,400 ^{QC,c}
357		Whole	26,500	23,800	25,150
357		Whole	31,500	29,300	30,400
418	51: 3	Lower half	69,100	---	69,100
418		Lower half	31,500	32,900	32,200
419		Lower half	36,400	32,700	34,550
419		Lower half	42,500	40,000	41,250
490	52: 3	Lower half	18,800	19,600	19,200
490		Lower half	19,900	19,500	19,700
491		Lower half	19,900	19,400	19,650
491		Lower half	22,300	22,300	22,300
549	Core 51 composite	Whole	33,400	32,900	33,150 ^{QC,d}
503	Core 52 composite	Whole	20,100	39,500	29,800 ^{QC,c}
Solids: fusion			µg/g	µg/g	µg/g
387	50: 1R	Subsegment	19,400	18,600	19,000
309	50: 2	Whole	20,300	20,400	20,350
322	50: 3	Whole	22,500	25,100	23,800
406	51: 2	Whole	34,600	38,700	36,650
255	51: 3	Lower half	34,200	34,300	34,250
254		Upper half	31,200	25,800	28,500
257	51: 4	Lower half	20,000	19,300	19,650
256		Upper half	34,000	36,100	35,050
455	52: 1	Whole	47,700	33,400	40,550 ^{QC,d}
481	52: 2	Whole	20,600	21,500	21,050
483	52: 3	Lower half	19,000	19,000	19,000
482		Upper half	24,500	22,300	23,400

Table B2-20. Tank 241-T-107 Analytical Results: Iron (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion (Cont'd)			µg/g	µg/g	µg/g
436	Core 51 composite	Whole	26,300	26,600	26,450
495	Core 52 composite	Whole	24,100	25,000	24,550
500		Whole	21,000	42,800	31,900 ^{QC:c}
Solids: water digest			µg/g	µg/g	µg/g
534	Core 51 composite	Whole	315	229	272 ^{QC:c}
495	Core 52 composite	Whole	429	449	439
Liquids			µg/mL	µg/mL	µg/mL
393	Core 50 composite	Drainable liquid	5.82	5.67	5.745
442	Core 51 composite	Drainable liquid	47.5	48.5	48
494	Core 52 composite	Drainable liquid	18.9	19.2	19.05

Table B2-21. Tank 241-T-107 Analytical Results: Lanthanum (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: acid digest			µg/g	µg/g	µg/g
356	50: 2	Whole	<2	<1.99	<1.995 ^{QC:r}
356		Whole	<1.65	<1.7	<1.675 ^{QC:e}
357		Whole	<2	<2	<2 ^{QC:e,r}
357		Whole	12.3	33.2	22.75 ^{QC:e}
418	51: 3	Lower half	<1.3	---	<1.3
418		Lower half	<1.99	<1.98	<1.985 ^{QC:c} .r
419		Lower half	<1.99	<1.98	<1.985 ^{QC:r}
419		Lower half	<1.69	<1.67	<1.68 ^{QC:c}
490	52: 3	Lower half	<1.7	<1.67	<1.685
490		Lower half	<1.69	<1.68	<1.685 ^{QC:c}
491		Lower half	<1.67	<1.69	<1.68
491		Lower half	<1.68	<1.68	<1.68 ^{QC:c}
549	Core 51 composite	Whole	<16.9	<16.9	<16.9
503	Core 52 composite	Whole	<2	<1.98	<1.99 ^{QC:e,r}
Solids: fusion			µg/g	µg/g	µg/g
387	50: 1R	Subsegment	<9.9	<9.88	<9.89
309	50: 2	Whole	<10	<9.94	<9.97
322	50: 3	Whole	<9.9	<9.9	<9.9
406	51: 2	Whole	<9.96	<9.96	<9.96
255	51: 3	Lower half	<9.75	<10	<9.875
254		Upper half	<9.4	<9.24	<9.32
257	51: 4	Lower half	<9.65	<9.6	<9.625
256		Upper half	<9.62	<9.73	<9.675
455	52: 1	Whole	<83.7	<83.5	<83.6
481	52: 2	Whole	<9.96	<9.96	<9.96

Table B2-21. Tank 241-T-107 Analytical Results: Lanthanum (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion (Cont'd)			µg/g	µg/g	µg/g
483	52: 3	Lower half	<9.94	<9.86	<9.9
482		Upper half	<9.92	<9.84	<9.88
436	Core 51 composite	Whole	<9.96	<9.9	<9.93
495	Core 52 composite	Whole	<83.8	<84	<83.9
500		Whole	<9.92	<9.98	<9.95
Solids: water digest			µg/g	µg/g	µg/g
534	Core 51 composite	Whole	<1.79	<1.79	<1.79
495	Core 52 composite	Whole	<17.2	<17.2	<17.2
Liquids			µg/mL	µg/mL	µg/mL
393	Core 50 composite	Drainable liquid	<0.2	<0.2	<0.2
442	Core 51 composite	Drainable liquid	<0.2	<0.2	<0.2
494	Core 52 composite	Drainable liquid	<0.500	<0.500	<0.500

Table B2-22. Tank 241-T-107 Analytical Results: Lead (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: acid digest			µg/g	µg/g	µg/g
356	50: 2	Whole	589	477	533 ^{QC:c}
356		Whole	763	534	648.5 ^{QC:d,c}
357		Whole	733	665	699 ^{QC:d}
357		Whole	695	632	663.5 ^{QC:c}
418	51: 3	Lower half	1,660	---	1,660
418		Lower half	1,030	1,080	1,055 ^{QC:d}
419		Lower half	1,100	1,000	1,050
419		Lower half	1,340	1,230	1,285 ^{QC:c}
490	52: 3	Lower half	139	137	138
490		Lower half	144	146	145
491		Lower half	144	143	143.5
491		Lower half	166	162	164
549	Core 51 composite	Whole	1,170	1,040	1,105
503	Core 52 composite	Whole	357	618	487.5 ^{QC:c}
Solids: fusion			µg/g	µg/g	µg/g
387	50: 1R	Subsegment	239	244	241.5
309	50: 2	Whole	535	525	530
322	50: 3	Whole	448	544	496
406	51: 2	Whole	1,340	1,320	1,330
255	51: 3	Lower half	895	1,080	987.5
254		Upper half	1,550	1,450	1,500
257	51: 4	Lower half	278	249	263.5
256		Upper half	965	1,040	1,002.5
455	52: 1	Whole	1,180	2,500	1,840 ^{QC:c}
481	52: 2	Whole	444	395	419.5
483	52: 3	Lower half	144	119	131.5
482		Upper half	491	412	451.5

Table B2-22. Tank 241-T-107 Analytical Results: Lead (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion (Cont'd)			µg/g	µg/g	µg/g
436	Core 51 composite	Whole	763	690	726.5
495	Core 52 composite	Whole	482	611	546.5 ^{QC:e}
500		Whole	346	796	571 ^{QC:e}
Solids: water digest			µg/g	µg/g	µg/g
534	Core 51 composite	Whole	20.3	20.1	20.2
495	Core 52 composite	Whole	< 62.8	< 62.8	< 62.8
Liquids			µg/mL	µg/mL	µg/mL
393	Core 50 composite	Drainable liquid	< 0.78	< 0.78	< 0.78
442	Core 51 composite	Drainable liquid	< 0.78	< 0.78	< 0.78
494	Core 52 composite	Drainable liquid	< 1.95	< 1.95	< 1.95

Table B2-23. Tank 241-T-107 Analytical Results: Lithium (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: acid digest			µg/g	µg/g	µg/g
356	50: 2	Whole	5.59	4.45	5.02 ^{QC:c}
356		Whole	7.47	5.09	6.28 ^{QC:c,c}
357		Whole	5.65	5.53	5.59
357		Whole	7.09	6.46	6.775
418	51: 3	Lower half	13	---	13
418		Lower half	7.28	7.9	7.59
419		Lower half	8.19	7.39	7.79
419		Lower half	9.79	9.1	9.445
490	52: 3	Lower half	1.22	1.33	1.275
490		Lower half	1.39	1.48	1.435
491		Lower half	1.34	1.5	1.42
491		Lower half	1.58	1.57	1.575
549	Core 51 composite	Whole	7.59	6.32	6.955 ^{QC:a}
503	Core 52 composite	Whole	3.97	4.05	4.01
Solids: fusion			µg/g	µg/g	µg/g
387	50: 1R	Subsegment	<2.97	<2.96	<2.965
309	50: 2	Whole	4.4	4.34	4.37
322	50: 3	Whole	3.31	4.69	4 ^{QC:c}
406	51: 2	Whole	8.79	8.75	8.77
255	51: 3	Lower half	8.64	7.03	7.835 ^{QC:c}
254		Upper half	7.74	6.86	7.3
257	51: 4	Lower half	<2.9	2.89	<2.895
256		Upper half	8.95	8.5	8.725
455	52: 1	Whole	<19.7	<19.6	<19.65
481	52: 2	Whole	6.21	5.8	6.005
483	52: 3	Lower half	5.55	4.57	5.06
482		Upper half	7.88	8.64	8.26

Table B2-23. Tank 241-T-107 Analytical Results: Lithium (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion (Cont'd)			µg/g	µg/g	µg/g
436	Core 51 composite	Whole	4.87	5.07	4.97
495	Core 52 composite	Whole	< 19.7	< 19.8	< 19.75
500		Whole	3.14	4.98	4.06 ^{QC}
Solids: water digest			µg/g	µg/g	µg/g
534	Core 51 composite	Whole	0.904	0.793	0.8485
495	Core 52 composite	Whole	< 4.05	< 4.05	< 4.05
Liquids			µg/mL	µg/mL	µg/mL
393	Core 50 composite	Drainable liquid	< 0.06	< 0.06	< 0.06
442	Core 51 composite	Drainable liquid	< 0.06	< 0.06	< 0.06
494	Core 52 composite	Drainable liquid	< 0.150	< 0.150	< 0.150

Table B2-24. Tank 241-T-107 Analytical Results: Magnesium (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: acid digest			µg/g	µg/g	µg/g
356	50: 2	Whole	282	194	238 ^{QC:c}
356		Whole	228	169	198.5 ^{QC:b,c}
357		Whole	211	215	213 ^{QC:b,d}
357		Whole	327	246	286.5 ^{QC:d,e}
418	51: 3	Lower half	323	---	323
418		Lower half	210	223	216.5 ^{QC:d}
419		Lower half	224	211	217.5
419		Lower half	273	255	264 ^{QC:c}
490	52: 3	Lower half	149	150	149.5
490		Lower half	153	148	150.5
491		Lower half	163	159	161
491		Lower half	163	172	167.5 ^{QC:c}
549	Core 51 composite	Whole	265	259	262
503	Core 52 composite	Whole	157	173	165
Solids: fusion			µg/g	µg/g	µg/g
387	50: 1R	Subsegment	180	174	177
309	50: 2	Whole	252	250	251
322	50: 3	Whole	211	234	222.5
406	51: 2	Whole	547	533	540
255	51: 3	Lower half	258	251	254.5
254		Upper half	286	242	264
257	51: 4	Lower half	214	227	220.5
256		Upper half	337	352	344.5
455	52: 1	Whole	407	443	425
481	52: 2	Whole	193	166	179.5
483	52: 3	Lower half	156	161	158.5
482		Upper half	202	192	197

Table B2-24. Tank 241-T-107 Analytical Results: Magnesium (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion (Cont'd)			µg/g	µg/g	µg/g
436	Core 51 composite	Whole	223	240	231.5
495	Core 52 composite	Whole	302	189	245.5 ^{QC:c}
500		Whole	190	252	221 ^{QC:c}
Solids: water digest			µg/g	µg/g	µg/g
534	Core 51 composite	Whole	7.3	12.2	9.75 ^{QC:c}
495	Core 52 composite	Whole	10.0	9.83	9.92
Liquids			µg/mL	µg/mL	µg/mL
393	Core 50 composite	Drainable liquid	1.32	1.16	1.24
442	Core 51 composite	Drainable liquid	0.511	0.31	0.4105 ^{QC:c}
494	Core 52 composite	Drainable liquid	0.390	0.396	0.393 ^{QC:b}

Table B2-25. Tank 241-T-107 Analytical Results: Manganese (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: acid digest			µg/g	µg/g	µg/g
356	50: 2	Whole	235	197	216
356		Whole	316	225	270.5 ^{QC:d,c}
357		Whole	263	249	256 ^{QC:c}
357		Whole	325	319	322 ^{QC:d}
418	51: 3	Lower half	434	---	434
418		Lower half	233	249	241 ^{QC:d}
419		Lower half	252	232	242
419		Lower half	323	300	311.5 ^{QC:c}
490	52: 3	Lower half	59.3	57.1	58.2
490		Lower half	56.4	60.1	58.25
491		Lower half	59.2	57	58.1
491		Lower half	68.3	65.5	66.9
549	Core 51 composite	Whole	236	226	231 ^{QC:a}
503	Core 52 composite	Whole	126	298	212 ^{QC:c}
Solids: fusion			µg/g	µg/g	µg/g
387	50: 1R	Subsegment	95.5	95.2	95.35
309	50: 2	Whole	242	229	235.5
322	50: 3	Whole	139	159	149
406	51: 2	Whole	824	933	878.5
255	51: 3	Lower half	278	285	281.5
254		Upper half	292	267	279.5
257	51: 4	Lower half	98.9	99.6	99.25
256		Upper half	250	268	259
455	52: 1	Whole	1,850	3,280	2,565 ^{QC:c}
481	52: 2	Whole	226	206	216
483	52: 3	Lower half	99.4	85.7	92.55
482		Upper half	175	158	166.5

Table B2-25. Tank 241-T-107 Analytical Results: Manganese (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion (Cont'd)			µg/g	µg/g	µg/g
436	Core 51 composite	Whole	196	183	189.5
495	Core 52 composite	Whole	197	177	187
500		Whole	161	313	237 ^{QC:c}
Solids: water digest			µg/g	µg/g	µg/g
534	Core 51 composite	Whole	1.78	2.37	2.075 ^{QC:c}
495	Core 52 composite	Whole	<3.04	3.04	<3.04
Liquids			µg/mL	µg/mL	µg/mL
393	Core 50 composite	Drainable liquid	<0.03	0.0345	<0.03225
442	Core 51 composite	Drainable liquid	0.0756	0.0811	0.07835
494	Core 52 composite	Drainable liquid	<0.0750	<0.0750	<0.0750

Table B2-26. Tank 241-T-107 Analytical Results: Molybdenum (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: acid digest			µg/g	µg/g	µg/g
356	50: 2	Whole	12	9.85	10.925
356		Whole	15.1	10.7	12.9 ^{QC:c}
357		Whole	13.5	13	13.25
357		Whole	10.5	10.8	10.65
418	51: 3	Lower half	12.9	---	12.9
418		Lower half	7.07	7.63	7.35
419		Lower half	7.63	6.77	7.2
419		Lower half	9.93	10.1	10.015
490	52: 3	Lower half	3.61	3.81	3.71
490		Lower half	3.73	3.56	3.645
491		Lower half	4.05	3.87	3.96
491		Lower half	3.53	3.24	3.385
549	Core 51 composite	Whole	<5.98	<5.95	<5.965
503	Core 52 composite	Whole	5.57	7.26	6.415 ^{QC:c}
Solids: fusion			µg/g	µg/g	µg/g
387	50: 1R	Subsegment	4.8	<4.45	<4.625
309	50: 2	Whole	12.9	12.8	12.85
322	50: 3	Whole	<4.46	4.49	<4.475
406	51: 2	Whole	20.6	18.2	19.4
255	51: 3	Lower half	9.53	11.7	10.615 ^{QC:c}
254		Upper half	12.2	12.5	12.35
257	51: 4	Lower half	5.27	7.3	6.285 ^{QC:c}
256		Upper half	7.13	6.93	7.03
455	52: 1	Whole	<29.5	<29.5	<29.5
481	52: 2	Whole	13.7	13.1	13.4
483	52: 3	Lower half	9.77	8.21	8.99
482		Upper half	9.82	5.74	7.78 ^{QC:c}

Table B2-26. Tank 241-T-107 Analytical Results: Molybdenum (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion (Cont'd)			µg/g	µg/g	µg/g
436	Core 51 composite	Whole	10.5	10.6	10.55
495	Core 52 composite	Whole	<29.6	<29.6	<29.6
500		Whole	7.96	6.63	7.295
Solids: water digest			µg/g	µg/g	µg/g
534	Core 51 composite	Whole	8.72	8.67	8.695
495	Core 52 composite	Whole	6.96	7.17	7.06
Liquids			µg/mL	µg/mL	µg/mL
393	Core 50 composite	Drainable liquid	2.23	2.14	2.185
442	Core 51 composite	Drainable liquid	22.2	22.6	22.4
494	Core 52 composite	Drainable liquid	2.09	2.16	2.125

Table B2-27. Tank 241-T-107 Analytical Results: Neodymium (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: acid digest			µg/g	µg/g	µg/g
356	50: 2	Whole	91.7	68	79.85 ^{QC:e}
356		Whole	71.7	52.5	62.1 ^{QC:e}
357		Whole	68.4	64.9	66.65
357		Whole	116	76.3	96.15 ^{QC:e}
418	51: 3	Lower half	185	---	185
418		Lower half	122	136	129
419		Lower half	136	118	127
419		Lower half	156	142	149 ^{QC:e}
490	52: 3	Lower half	8.91	< 8.1	< 8.505
490		Lower half	13.9	< 8.07	< 10.985 ^{QC:e}
491		Lower half	14	16	15
491		Lower half	< 8.1	9.39	< 8.745
549	Core 51 composite	Whole	106	112	109
503	Core 52 composite	Whole	35.5	38.2	36.85
Solids: fusion			µg/g	µg/g	µg/g
387	50: 1R	Subsegment	< 38.6	< 38.5	< 38.55
309	50: 2	Whole	40.2	73.5	56.85 ^{QC:e}
322	50: 3	Whole	76.9	84.3	80.6
406	51: 2	Whole	107	79.1	93.05 ^{QC:e}
255	51: 3	Lower half	198	140	169 ^{QC:e}
254		Upper half	117	120	118.5
257	51: 4	Lower half	< 37.6	71.2	< 54.4 ^{QC:e}
256		Upper half	147	136	141.5
455	52: 1	Whole	< 404	< 403	< 403.5
481	52: 2	Whole	76.8	< 38.8	< 57.8 ^{QC:e}
483	52: 3	Lower half	70.7	49.4	60.05 ^{QC:e}
482		Upper half	59.7	96.3	78 ^{QC:e}

Table B2-27. Tank 241-T-107 Analytical Results: Neodymium (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion (Cont'd)			µg/g	µg/g	µg/g
436	Core 51 composite	Whole	81.1	83.3	82.2
495	Core 52 composite	Whole	< 404	< 405	< 404.5
500		Whole	55.3	< 38.9	< 47.1 ^{QCc}
Solids: water digest			µg/g	µg/g	µg/g
534	Core 51 composite	Whole	< 8.63	< 8.63	< 8.63
495	Core 52 composite	Whole	< 83.0	< 83.1	< 83.1
Liquids			µg/mL	µg/mL	µg/mL
393	Core 50 composite	Drainable liquid	0.949	1.12	1.0345
442	Core 51 composite	Drainable liquid	0.795	1.04	0.9175 ^{QCc}
494	Core 52 composite	Drainable liquid	< 1.95	< 1.95	< 1.95

Table B2-28. Tank 241-T-107 Analytical Results: Nickel (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: acid digest			µg/g	µg/g	µg/g
356	50: 2	Whole	137	115	126
356		Whole	177	126	151.5 ^{QC:c}
357		Whole	177	172	174.5
357		Whole	137	142	139.5
418	51: 3	Lower half	299	---	299
418		Lower half	178	192	185
419		Lower half	193	178	185.5
419		Lower half	244	226	235 ^{QC:c}
490	52: 3	Lower half	35.2	34.4	34.8
490		Lower half	33.6	33.9	33.75
491		Lower half	35.1	34.5	34.8
491		Lower half	38.3	37.6	37.95
549	Core 51 composite	Whole	308	301	304.5 ^{QC:a}
503	Core 52 composite	Whole	274	285	279.5
Solids: fusion			µg/g	µg/g	µg/g
387	50: 1R	Subsegment	9,780	8,070	8,925 ^{QC:r}
309	50: 2	Whole	4,470	3,470	3,970 ^{QC:e,r}
322	50: 3	Whole	4,480	4,670	4,575 ^{QC:r}
406	51: 2	Whole	11,900	12,300	12,100 ^{QC:r}
255	51: 3	Lower half	1,070	637	853.5 ^{QC:e,r}
254		Upper half	1,700	968	1,334 ^{QC:e,r}
257	51: 4	Lower half	10,200	8,490	9,345 ^{QC:r}
256		Upper half	983	738	860.5 ^{QC:e,r}
481	52: 2	Whole	2,610	3,220	2,915 ^{QC:e,r}
483	52: 3	Lower half	8,130	10,700	9,415 ^{QC:e,r}
482		Upper half	3,440	1,060	2,250 ^{QC:e,r}
436	Core 51 composite	Whole	2,850	1,370	2,110 ^{QC:e,r}
Solids: fusion			µg/g	µg/g	µg/g
500	Core 52 composite	Whole	3,800	2,530	3,165 ^{QC:e,r}

Table B2-28. Tank 241-T-107 Analytical Results: Nickel (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: water digest			µg/g	µg/g	µg/g
534	Core 51 composite	Whole	5.18	3.79	4.485 ^{QC-c}
495	Core 52 composite	Whole	< 13.2	< 13.2	< 13.2
Liquids			µg/mL	µg/mL	µg/mL
393	Core 50 composite	Drainable liquid	1.79	1.75	1.77
442	Core 51 composite	Drainable liquid	15.9	16.2	16.05
494	Core 52 composite	Drainable liquid	2.75	2.82	2.785

Note:

The fusion results for nickel are not valid because of contamination from the nickel crucible during the fusion.

Table B2-29. Tank 241-T-107 Analytical Results: Phosphorus (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: acid digest			µg/g	µg/g	µg/g
356	50: 2	Whole	4,440	14,200	9,320 ^{QC,c}
356		Whole	8,170	4,050	6,110 ^{QC,c,e}
357		Whole	4,850	4,470	4,660
357		Whole	4,120	5,790	4,955 ^{QC,d,e}
418	51: 3	Lower half	6,330	---	6,330
418		Lower half	13,400	9,040	11,220 ^{QC,c}
419		Lower half	7,460	9,840	8,650 ^{QC,c}
419		Lower half	4,750	11,500	8,125 ^{QC,c}
490	52: 3	Lower half	24,800	28,000	26,400
490		Lower half	28,000	24,200	26,100
491		Lower half	26,000	27,000	26,500
491		Lower half	27,600	27,400	27,500
549	Core 51 composite	Whole	33,900	25,400	29,650 ^{QC,c,e}
503	Core 52 composite	Whole	33,400	27,300	30,350
Solids: fusion			µg/g	µg/g	µg/g
387	50: 1R	Subsegment	28,700	32,400	30,550
309	50: 2	Whole	4,340	3,340	3,840 ^{QC,c}
322	50: 3	Whole	44,500	40,800	42,650
406	51: 2	Whole	5,350	5,310	5,330
255	51: 3	Lower half	7,880	7,350	7,615
254		Upper half	21,300	28,900	25,100 ^{QC,c}
257	51: 4	Lower half	31,000	33,900	32,450
256		Upper half	12,700	6,730	9,715 ^{QC,c}
455	52: 1	Whole	<226	<226	<226
481	52: 2	Whole	22,200	29,100	25,650 ^{QC,c}
483	52: 3	Lower half	25,500	26,400	25,950
482		Upper half	38,700	35,000	36,850

Table B2-29. Tank 241-T-107 Analytical Results: Phosphorus (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion (Cont'd)			µg/g	µg/g	µg/g
436	Core 51 composite	Whole	31,500	34,300	32,900
495	Core 52 composite	Whole	34,500	34,500	34,500 ^{QC:d}
500		Whole	33,600	28,900	31,250
Solids: water digest			µg/g	µg/g	µg/g
534	Core 51 composite	Whole	27,300	33,300	30,300 ^{QC:d}
495	Core 52 composite	Whole	18,400	15,900	17,100 ^{QC:d}
Liquids			µg/mL	µg/mL	µg/mL
393	Core 50 composite	Drainable liquid	785	795	790
442	Core 51 composite	Drainable liquid	2,020	2,030	2,025
494	Core 52 composite	Drainable liquid	2,630	2,550	2,590

Table B2-30. Tank 241-T-107 Analytical Results: Potassium (ICP).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: acid digest			µg/g	µg/g	µg/g
356	50: 2	Whole	235	195	215 ^{QC:b}
356		Whole	281	177	229 ^{QC:c}
357		Whole	251	227	239
357		Whole	252	242	247 ^{QC:b,c}
418	51: 3	Lower half	435	---	435
418		Lower half	295	311	303 ^{QC:d}
419		Lower half	316	291	303.5
419		Lower half	353	337	345 ^{QC:c}
490	52: 3	Lower half	218	239	228.5
490		Lower half	227	211	219 ^{QC:c}
491		Lower half	225	209	217
491		Lower half	210	217	213.5 ^{QC:d}
549	Core 51 composite	Whole	267	220	243.5 ^{QC:a}
503	Core 52 composite	Whole	233	217	225 ^{QC:b,c}
Solids: water digest			µg/g	µg/g	µg/g
534	Core 51 composite	Whole	641	260	450.5 ^{QC:c}
495	Core 52 composite	Whole	226	135	180
Liquids			µg/mL	µg/mL	µg/mL
393	Core 50 composite	Drainable liquid	46.8	46.6	46.7
442	Core 51 composite	Drainable liquid	360	374	367 ^{QC:d}
494	Core 52 composite	Drainable liquid	137	139	138 ^{QC:d}

Table B2-31. Tank 241-T-107 Analytical Results: Samarium (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: acid digest			µg/g	µg/g	µg/g
356	50: 2	Whole	144	119	131.5
356		Whole	376	203	289.5 ^{QC:a,c,e}
357		Whole	125	150	137.5
357		Whole	33.7	334	183.85 ^{QC:a,d}
418	51: 3	Lower half	1,120	---	1,120
418		Lower half	373	394	383.5 ^{QC:d}
419		Lower half	418	362	390
419		Lower half	923	886	904.5 ^{QC:c}
490	52: 3	Lower half	28.5	27.5	28 ^{QC:c}
490		Lower half	13.7	21.3	17.5 ^{QC:c}
491		Lower half	34.4	29.6	32
491		Lower half	36.4	37.9	37.15
549	Core 51 composite	Whole	480	448	464 ^{QC:a}
503	Core 52 composite	Whole	115	96.3	105.65 ^{QC:a,d,r}
Solids: fusion			µg/g	µg/g	µg/g
387	50: 1R	Subsegment	69.5	69.9	69.7
309	50: 2	Whole	195	194	194.5
322	50: 3	Whole	202	136	169 ^{QC:c}
406	51: 2	Whole	135	97.4	116.2 ^{QC:c}
255	51: 3	Lower half	305	< 52	< 178.5 ^{QC:c}
254		Upper half	239	103	171 ^{QC:c}
257	51: 4	Lower half	< 50.2	62.4	< 56.3
256		Upper half	194	235	214.5
455	52: 1	Whole	< 463	< 462	< 462.5 ^{QC:c}
481	52: 2	Whole	110	97.1	103.55
483	52: 3	Lower half	< 51.7	< 51.3	< 51.5
482		Upper half	106	102	104

Table B2-31. Tank 241-T-107 Analytical Results: Samarium (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion (Cont'd)			$\mu\text{g/g}$	$\mu\text{g/g}$	$\mu\text{g/g}$
436	Core 51 composite	Whole	148	208	178 ^{QC:c}
495	Core 52 composite	Whole	<464	<464	<464
500		Whole	79.5	63.4	71.45 ^{QC:c}
Solids: water digest			$\mu\text{g/g}$	$\mu\text{g/g}$	$\mu\text{g/g}$
534	Core 51 composite	Whole	<9.89	<9.89	<9.89
495	Core 52 composite	Whole	<95.1	<95.2	95.2
Liquids			$\mu\text{g/mL}$	$\mu\text{g/mL}$	$\mu\text{g/mL}$
393	Core 50 composite	Drainable liquid	2.07	<1.04	<1.555 ^{QC:c}
442	Core 51 composite	Drainable liquid	3.4	3.92	3.66
494	Core 52 composite	Drainable liquid	<2.6	<2.6	<2.6

Table B2-32. Tank 241-T-107 Analytical Results: Selenium (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: acid digest			µg/g	µg/g	µg/g
356	50: 2	Whole	99.1	67.8	83.45 ^{QC:c}
356		Whole	< 14.1	< 14	< 14.05 ^{QC:a,r}
357		Whole	97.1	85.3	91.2
357		Whole	< 14.1	< 14.1	< 14.1 ^{QC:a,c,r}
418	51: 3	Lower half	150	---	150
418		Lower half	< 14.1	< 14	< 14.05 ^{QC:a,c,r}
419		Lower half	122	114	118 ^{QC:c}
419		Lower half	< 14.1	< 13.9	< 14 ^{QC:a,r}
490	52: 3	Lower half	138	134	136
490		Lower half	< 8.63	< 8.6	< 8.615 ^{QC:c,r}
491		Lower half	157	150	153.5 ^{QC:c}
491		Lower half	< 8.6	< 8.6	< 8.6 ^{QC:c,r}
549	Core 51 composite	Whole	104	< 86.3	< 95.15 ^{QC:a}
503	Core 52 composite	Whole	< 14.1	< 14	< 14.05 ^{QC:a,c,r}
Solids: fusion			µg/g	µg/g	µg/g
387	50: 1R	Subsegment	< 69.8	< 69.7	< 69.75
309	50: 2	Whole	< 70.5	< 70.1	< 70.3 ^{QC:r}
322	50: 3	Whole	< 69.8	< 69.8	< 69.8 ^{QC:c,r}
406	51: 2	Whole	< 70.2	< 70.2	< 70.2
255	51: 3	Lower half	< 68.7	< 70.5	< 69.6
254		Upper half	< 66.3	< 65.2	< 65.75
257	51: 4	Lower half	< 68.1	< 67.7	< 67.9
256		Upper half	< 67.8	< 68.6	< 68.2
455	52: 1	Whole	< 428	< 427	< 427.5
481	52: 2	Whole	< 70.2	< 70.2	< 70.2
483	52: 3	Lower half	< 70.1	< 69.5	< 69.8
482		Upper half	< 69.9	< 69.4	< 69.65

Table B2-32. Tank 241-T-107 Analytical Results: Selenium (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion (Cont'd)			µg/g	µg/g	µg/g
436	Core 51 composite	Whole	< 70.2	< 69.8	< 70 ^{QC:c}
495	Core 52 composite	Whole	< 429	< 430	< 429.5
500		Whole	< 69.9	< 70.4	< 70.15 ^{QC:c}
Solids: water digest			µg/g	µg/g	µg/g
534	Core 51 composite	Whole	58.6	52.1	55.35 ^{QC:a}
495	Core 52 composite	Whole	< 88.1	89.4	< 88.8
Liquids			µg/mL	µg/mL	µg/mL
393	Core 50 composite	Drainable liquid	< 1.41	< 1.41	< 1.41 ^{QC:a,c,r}
442	Core 51 composite	Drainable liquid	< 1.41	< 1.41	< 1.41 ^{QC:a,c,r}
494	Core 52 composite	Drainable liquid	< 3.52	< 3.52	< 3.52 ^{QC:a,c,r}

Table B2-33. Tank 241-T-107 Analytical Results: Silicon (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: acid digest			µg/g	µg/g	µg/g
356	50: 2	Whole	1,600	1,200	1,400 ^{QC:b,c,r}
356		Whole	1,810	1,320	1,565 ^{QC:b,d,e}
357		Whole	1,550	1,690	1,620 ^{QC:b,c,r}
357		Whole	2,140	1,950	2,045 ^{QC:b,d}
418	51: 3	Lower half	794	---	794 ^{QC:b,r}
418		Lower half	< 3.39	< 3.37	< 3.38 ^{QC:a,c,r}
419		Lower half	1,100	544	822 ^{QC:b,c,e,r}
419		Lower half	< 3.39	< 3.36	< 3.375 ^{QC:a,r}
490	52: 3	Lower half	836	676	756 ^{QC:b,c,e,r}
490		Lower half	2,050	2,350	2,200 ^{QC:b,c,r}
491		Lower half	901	784	842.5 ^{QC:b,c,r}
491		Lower half	1,660	1,430	1,545 ^{QC:b,d,r}
549	Core 51 composite	Whole	63.7	156	109.85 ^{QC:b,e}
503	Core 52 composite	Whole	1,820	1,460	1,640 ^{QC:b,c,e,r}
Solids: fusion			µg/g	µg/g	µg/g
387	50: 1R	Subsegment	9,970	9,720	9,845
309	50: 2	Whole	2,230	1,880	2,055
322	50: 3	Whole	4,670	5,250	4,960
406	51: 2	Whole	5,740	4,440	5,090 ^{QC:c}
255	51: 3	Lower half	925	1,140	1,032.5 ^{QC:c}
254		Upper half	1,100	863	981.5 ^{QC:c}
257	51: 4	Lower half	10,100	9,870	9,985
256		Upper half	2,680	2,780	2,730
455	52: 1	Whole	4,600	4,720	4,660 ^{QC:d,r}
481	52: 2	Whole	5,890	5,190	5,540
483	52: 3	Lower half	11,400	11,100	11,250
482		Upper half	4,680	5,090	4,885

Table B2-33. Tank 241-T-107 Analytical Results: Silicon (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion (Cont'd)			µg/g	µg/g	µg/g
436	Core 51 composite	Whole	4,980	4,750	4,865
495	Core 52 composite	Whole	7,260	7,370	7,315 ^{QC:d,r}
500		Whole	7,110	7,390	7,250
Solids: water digest			µg/g	µg/g	µg/g
534	Core 51 composite	Whole	3,710	4,180	3,945
495	Core 52 composite	Whole	<16.2	<16.2	<16.2
Liquids			µg/mL	µg/mL	µg/mL
393	Core 50 composite	Drainable liquid	60.8	56.2	58.5 ^{QC:b,d,r}
442	Core 51 composite	Drainable liquid	65.8	66.8	66.3 ^{QC:b,d,r}
494	Core 52 composite	Drainable liquid	78.2	96.1	87.15 ^{QC:b,d,r}

Table B2-34. Tank 241-T-107 Analytical Results: Silver (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: acid digest			µg/g	µg/g	µg/g
356	50: 2	Whole	2.99	2.44	2.715 ^{QC:a,r}
356		Whole	5.69	8.22	6.955 ^{QC:e}
357		Whole	32.7	64.1	48.4 ^{QC:e}
357		Whole	2.21	1.7	1.955 ^{QC:a,e,c,r}
418	51: 3	Lower half	16.1	---	16.1 ^{QC:a,r}
418		Lower half	<0.897	<0.891	<0.894 ^{QC:a,c,r}
419		Lower half	19.3	16.2	17.75 ^{QC:a,c,r}
419		Lower half	<0.897	<0.889	<0.893 ^{QC:a,r}
490	52: 3	Lower half	<0.496	<0.494	<0.495 ^{QC:c}
490		Lower half	<0.5	<0.492	<0.496 ^{QC:a,c,r}
491		Lower half	<0.494	<0.494	<0.494
491		Lower half	<0.492	<0.498	<0.495 ^{QC:a,c,r}
549	Core 51 composite	Whole	6.79	7.94	7.365 ^{QC:a}
503	Core 52 composite	Whole	<0.898	<0.891	<0.8945 ^{QC:c}
Solids: fusion			µg/g	µg/g	µg/g
387	50: 1R	Subsegment	<4.46	<4.45	<4.455
309	50: 2	Whole	<4.5	<4.47	<4.485
322	50: 3	Whole	<4.46	<4.46	<4.46
406	51: 2	Whole	4.57	<4.48	<4.525
255	51: 3	Lower half	<4.39	<4.5	<4.445
254		Upper half	<4.23	<4.16	<4.195
257	51: 4	Lower half	<4.34	<4.32	<4.33
256		Upper half	6.21	<4.38	<5.295 ^{QC:e}
455	52: 1	Whole	<24.6	<24.6	<24.6
481	52: 2	Whole	<4.48	<4.48	<4.48

Table B2-34. Tank 241-T-107 Analytical Results: Silver (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion (Cont'd)			µg/g	µg/g	µg/g
483	52: 3	Lower half	5.31	5.09	5.2
482		Upper half	4.72	5.45	5.085
436	Core 51 composite	Whole	<4.48	<4.46	<4.47
495	Core 52 composite	Whole	<24.7	<24.7	<24.7
500		Whole	<4.46	<4.49	<4.475
Solids: water digest			µg/g	µg/g	µg/g
534	Core 51 composite	Whole	<0.526	<0.526	<0.526
495	Core 52 composite	Whole	<5.06	<5.09	<5.07
Liquids			µg/mL	µg/mL	µg/mL
393	Core 50 composite	Drainable liquid	<0.09	<0.09	<0.09 ^{QC:a}
442	Core 51 composite	Drainable liquid	0.254	0.262	0.258 ^{QC:a,c}
494	Core 52 composite	Drainable liquid	<0.225	<0.225	<0.225

Table B2-35. Tank 241-T-107 Analytical Results: Sodium (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: acid digest			µg/g	µg/g	µg/g
356	50: 2	Whole	61,800	76,400	69,100 ^{QC:b,e}
356		Whole	89,300	63,200	76,250 ^{QC:b,e}
357		Whole	82,600	76,800	79,700 ^{QC:b}
357		Whole	59,700	60,700	60,200 ^{QC:b,d}
418	51: 3	Lower half	1.130E+05	---	1.130E+05 ^{QC:b}
418		Lower half	92,300	84,800	88,550 ^{QC:b}
419		Lower half	75,600	88,700	82,150 ^{QC:b}
419		Lower half	95,700	98,900	97,300 ^{QC:b}
490	52: 3	Lower half	1.040E+05	1.090E+05	1.065E+05 ^{QC:b}
490		Lower half	1.160E+05	1.080E+05	1.120E+05 ^{QC:b}
491		Lower half	1.080E+05	1.080E+05	1.080E+05 ^{QC:b}
491		Lower half	1.250E+05	1.250E+05	1.250E+05 ^{QC:b}
549	Core 51 composite	Whole	1.420E+05	1.310E+05	1.365E+05 ^{QC:b,d}
503	Core 52 composite	Whole	1.310E+05	1.170E+05	1.240E+05 ^{QC:b}
Solids: fusion			µg/g	µg/g	µg/g
387	50: 1R	Subsegment	1.250E+05	1.300E+05	1.275E+05
309	50: 2	Whole	58,300	52,100	55,200 ^{QC:r}
322	50: 3	Whole	1.240E+05	1.210E+05	1.225E+05 ^{QC:e,r}
406	51: 2	Whole	72,000	70,200	71,100
255	51: 3	Lower half	78,900	76,900	77,900
254		Upper half	1.020E+05	1.140E+05	1.080E+05
257	51: 4	Lower half	1.200E+05	1.240E+05	1.220E+05
256		Upper half	88,000	76,900	82,450
455	52: 1	Whole	26,500	28,000	27,250 ^{QC:d}
481	52: 2	Whole	98,500	1.120E+05	1.052E+05
483	52: 3	Lower half	1.080E+05	1.060E+05	1.070E+05
482		Upper half	1.350E+05	1.270E+05	1.310E+05

Table B2-35. Tank 241-T-107 Analytical Results: Sodium (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion (Cont'd)			µg/g	µg/g	µg/g
436	Core 51 composite	Whole	1.190E+05	1.270E+05	1.230E+05
495	Core 52 composite	Whole	1.300E+05	1.400E+05	1.350E+05
500		Whole	1.150E+05	1.080E+05	1.115E+05 ^{QC:c}
Solids: water digest			µg/g	µg/g	µg/g
534	Core 51 composite	Whole	1.280E+05	1.400E+05	1.340E+05
495	Core 52 composite	Whole	87,100	76,500	81,800 ^{QC:d}
Liquids			µg/mL	µg/mL	µg/mL
393	Core 50 composite	Drainable liquid	14,600	14,600	14,600 ^{QC:b}
442	Core 51 composite	Drainable liquid	96,300	94,700	95,500 ^{QC:b}
494	Core 52 composite	Drainable liquid	52,300	51,500	51,900 ^{QC:b}

Table B2-36. Tank 241-T-107 Analytical Results: Strontium (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: acid digest			µg/g	µg/g	µg/g
356	50: 2	Whole	995	788	891.5 ^{QC:c}
356		Whole	1,210	832	1,021 ^{QC:d,e}
357		Whole	938	988	963
357		Whole	1,200	1,140	1,170 ^{QC:d}
418	51: 3	Lower half	2,030	---	2,030
418		Lower half	1,280	1,380	1,330 ^{QC:d}
419		Lower half	1,420	1,290	1,355
419		Lower half	1,710	1,600	1,655 ^{QC:c}
490	52: 3	Lower half	357	276	316.5
490		Lower half	390	329	359.5 ^{QC:d}
491		Lower half	290	360	325 ^{QC:d,e}
491		Lower half	335	319	327 ^{QC:d}
549	Core 51 composite	Whole	1,250	1,230	1,240
503	Core 52 composite	Whole	704	665	684.5 ^{QC:c}
Solids: fusion			µg/g	µg/g	µg/g
387	50: 1R	Subsegment	429	414	421.5
309	50: 2	Whole	853	841	847
322	50: 3	Whole	1,020	1,140	1,080
406	51: 2	Whole	1,060	1,020	1,040
255	51: 3	Lower half	1,350	1,350	1,350
254		Upper half	1,170	1,030	1,100
257	51: 4	Lower half	421	409	415
256		Upper half	1,560	1,650	1,605
455	52: 1	Whole	46.7	50.6	48.65
481	52: 2	Whole	681	616	648.5
483	52: 3	Lower half	372	291	331.5 ^{QC:c}
482		Upper half	960	930	945

Table B2-36. Tank 241-T-107 Analytical Results: Strontium (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion (Cont'd)			µg/g	µg/g	µg/g
436	Core 51 composite	Whole	934	974	954
495	Core 52 composite	Whole	652	814	733 ^{QC:c}
500		Whole	751	854	802.5
Solids: water digest			µg/g	µg/g	µg/g
534	Core 51 composite	Whole	6.44	4.54	5.49 ^{QC:c}
495	Core 52 composite	Whole	5.06	6.08	5.57
Liquids			µg/mL	µg/mL	µg/mL
393	Core 50 composite	Drainable liquid	0.14	0.139	0.1395
442	Core 51 composite	Drainable liquid	0.175	0.169	0.172
494	Core 52 composite	Drainable liquid	1.80	1.82	1.81

Table B2-37. Tank 241-T-107 Analytical Results: Sulfur (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: acid digest			µg/g	µg/g	µg/g
356	50: 2	Whole	3,110	2,580	2,845
356		Whole	4,320	3,420	3,870 ^{QC,c}
357		Whole	3,010	3,130	3,070 ^{QC,c}
357		Whole	4,490	4,860	4,675 ^{QC,c}
418	51: 3	Lower half	5,920	---	5,920
418		Lower half	4,030	4,260	4,145 ^{QC,d}
419		Lower half	4,390	4,060	4,225
419		Lower half	4,920	4,510	4,715 ^{QC,c}
490	52: 3	Lower half	3,430	3,510	3,470
490		Lower half	3,690	3,520	3,605 ^{QC,c}
491		Lower half	3,840	3,840	3,840
491		Lower half	3,680	3,470	3,575 ^{QC,c}
549	Core 51 composite	Whole	3,600	3,520	3,560 ^{QC,c}
503	Core 52 composite	Whole	2,490	2,570	2,530 ^{QC,d}
Solids: fusion			µg/g	µg/g	µg/g
387	50: 1R	Subsegment	2,960	2,880	2,920
309	50: 2	Whole	2,950	2,710	2,830
322	50: 3	Whole	1,290	1,470	1,380
406	51: 2	Whole	4,350	4,230	4,290
255	51: 3	Lower half	4,260	4,270	4,265
254		Upper half	3,640	3,250	3,445
257	51: 4	Lower half	3,110	3,060	3,085
256		Upper half	4,260	4,470	4,365
455	52: 1	Whole	1,080	1,150	1,115
481	52: 2	Whole	2,640	2,510	2,575
483	52: 3	Lower half	3,610	3,530	3,570
482		Upper half	2,470	2,630	2,550

Table B2-37. Tank 241-T-107 Analytical Results: Sulfur (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion (Cont'd)			µg/g	µg/g	µg/g
436	Core 51 composite	Whole	3,490	3,640	3,565
495	Core 52 composite	Whole	2,470	2,440	2,455
500		Whole	2,910	3,060	2,985
Solids: water digest			µg/g	µg/g	µg/g
534	Core 51 composite	Whole	4,100	3,840	3,970
495	Core 52 composite	Whole	3,360	2,860	3,110
Liquids			µg/mL	µg/mL	µg/mL
393	Core 50 composite	Drainable liquid	831	818	824.5 ^{QC,c,r}
442	Core 51 composite	Drainable liquid	6,030	5,990	6,010
494	Core 52 composite	Drainable liquid	3,350	3,320	3,335

Table B2-38. Tank 241-T-107 Analytical Results: Thallium (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: acid digest			µg/g	µg/g	µg/g
356	50: 2	Whole	44.8	39.3	42.05 ^{QC:a}
356		Whole	152	113	132.5 ^{QC:a,c,e}
357		Whole	327	584	455.5 ^{QC:a,c,e}
357		Whole	<25	67.3	<46.15 ^{QC:a,d,e}
418	51: 3	Lower half	403	---	403 ^{QC:a}
418		Lower half	205	232	218.5 ^{QC:d}
419		Lower half	219	201	210
419		Lower half	360	337	348.5 ^{QC:a,c}
490	52: 3	Lower half	27.9	26.3	27.1 ^{QC:a}
490		Lower half	<16.3	<16.2	<16.25
491		Lower half	26.9	<16.2	<21.55 ^{QC:c,e}
491		Lower half	<16.1	21.5	<18.8 ^{QC:a,d,e}
549	Core 51 composite	Whole	<164	<163	<163.5 ^{QC:a}
503	Core 52 composite	Whole	72	35.4	53.7 ^{QC:a,d,e,r}
Solids: fusion			µg/g	µg/g	µg/g
387	50: 1R	Subsegment	<124	<124	<124
309	50: 2	Whole	<125	<124	<124.5 ^{QC:r}
322	50: 3	Whole	203	<124	<163.5 ^{QC:c,r}
406	51: 2	Whole	<125	<125	<125
255	51: 3	Lower half	<122	<125	<123.5
254		Upper half	<167	<116	<141.5 ^{QC:c}
257	51: 4	Lower half	<121	<120	<120.5
256		Upper half	<120	<122	<121
455	52: 1	Whole	<807	<806	<806.5
481	52: 2	Whole	<125	<125	<125
483	52: 3	Lower half	<124	<123	<123.5
482		Upper half	<124	<123	<123.5

Table B2-38. Tank 241-T-107 Analytical Results: Thallium (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion (Cont'd)			$\mu\text{g/g}$	$\mu\text{g/g}$	$\mu\text{g/g}$
436	Core 51 composite	Whole	< 125	144	< 134.5
495	Core 52 composite	Whole	< 809	< 810	< 809.5
500		Whole	< 124	< 125	< 124.5
Solids: water digest			$\mu\text{g/g}$	$\mu\text{g/g}$	$\mu\text{g/g}$
534	Core 51 composite	Whole	< 17.3	< 17.3	< 17.3
495	Core 52 composite	Whole	< 166	< 166	< 166
Liquids			$\mu\text{g/mL}$	$\mu\text{g/mL}$	$\mu\text{g/mL}$
393	Core 50 composite	Drainable liquid	< 2.5	< 2.5	< 2.5
442	Core 51 composite	Drainable liquid	< 2.5	3.21	< 2.855 ^{QC,d}
494	Core 52 composite	Drainable liquid	< 6.25	< 6.25	< 6.25

Table B2-39. Tank 241-T-107 Analytical Results: Titanium (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: acid digest			µg/g	µg/g	µg/g
356	50: 2	Whole	<0.699	<0.697	<0.698
356		Whole	<0.292	5.68	<2.986 ^{QC:c}
357		Whole	8.49	19.7	14.095 ^{QC:c}
357		Whole	<0.699	<0.699	<0.699
418	51: 3	Lower half	<0.23	---	<0.23
418		Lower half	<0.698	<0.693	<0.6955 ^{QC:c}
419		Lower half	<0.698	<0.692	<0.695
419		Lower half	<0.299	<0.295	<0.297 ^{QC:c}
490	52: 3	Lower half	2.47	2.44	2.455
490		Lower half	2.61	2.04	2.325 ^{QC:c}
491		Lower half	2.32	2.66	2.49
491		Lower half	2.67	2.28	2.475
549	Core 51 composite	Whole	<2.99	<2.98	<2.985
503	Core 52 composite	Whole	4.11	146	75.055 ^{QC:c,r}
Solids: fusion			µg/g	µg/g	µg/g
387	50: 1R	Subsegment	<3.47	<3.46	<3.465
309	50: 2	Whole	39.8	33.7	36.75
322	50: 3	Whole	<3.47	31.4	<17.435 ^{QC:c}
406	51: 2	Whole	97.5	76.1	86.8 ^{QC:c}
255	51: 3	Lower half	<3.41	<3.5	<3.455
254		Upper half	<23	22	<22.5
257	51: 4	Lower half	<3.38	4.51	<3.945 ^{QC:c}
256		Upper half	<3.37	<3.4	<3.385
455	52: 1	Whole	148	275	211.5 ^{QC:c}
481	52: 2	Whole	22.4	7.89	15.145 ^{QC:c}
483	52: 3	Lower half	8.96	13.9	11.43 ^{QC:c}
482		Upper half	4.89	4.3	4.595

Table B2-39. Tank 241-T-107 Analytical Results: Titanium (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion (Cont'd)			µg/g	µg/g	µg/g
436	Core 51 composite	Whole	<3.49	16.3	<9.895 ^{QC:c}
495	Core 52 composite	Whole	19.2	<14.8	<17
500		Whole	5.22	7.28	6.25 ^{QC:c}
Solids: water digest			µg/g	µg/g	µg/g
534	Core 51 composite	Whole	1.25	2.98	2.115 ^{QC:c}
495	Core 52 composite	Whole	<3.04	<3.04	<3.04
Liquids			µg/mL	µg/mL	µg/mL
393	Core 50 composite	Drainable liquid	<0.07	<0.07	<0.07
442	Core 51 composite	Drainable liquid	<0.07	<0.07	<0.07
494	Core 52 composite	Drainable liquid	<0.175	<0.175	<0.175

Table B2-40. Tank 241-T-107 Analytical Results: Zirconium (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: acid digest			µg/g	µg/g	µg/g
356	50: 2	Whole	16.0	18.3	17.15 ^{QC:c}
356		Whole	<1.2	<1.2	<1.2 ^{QC:r}
357		Whole	49.6	64.8	57.2 ^{QC:c,e}
357		Whole	<1.2	<1.2	<1.2 ^{QC:c,r}
418	51: 3	Lower half	50.9	---	50.9
418		Lower half	13.5	13.9	13.7
419		Lower half	15.5	13.3	14.4
419		Lower half	42.1	39.2	40.65
490	52: 3	Lower half	49.1	56.5	52.8
490		Lower half	29.3	40.2	34.75 ^{QC:c,e,r}
491		Lower half	24.1	19.9	22 ^{QC:c,r}
491		Lower half	50.4	55.6	53.0
549	Core 51 composite	Whole	117	121	119
503	Core 52 composite	Whole	22.5	25.7	24.1 ^{QC:c,r}
Solids: fusion			µg/g	µg/g	µg/g
387	50: 1R	Subsegment	12.4	30.2	21.3 ^{QC:c}
309	50: 2	Whole	<6	<5.96	<5.98
322	50: 3	Whole	<5.94	<5.94	<5.94
406	51: 2	Whole	<5.98	48.2	<27.09 ^{QC:c}
255	51: 3	Lower half	11	<6	<8.5 ^{QC:c}
254		Upper half	19.4	11.9	15.65 ^{QC:c}
257	51: 4	Lower half	17.9	35.9	26.9 ^{QC:c}
256		Upper half	45.8	45.6	45.7
455	52: 1	Whole	97.2	178	137.6 ^{QC:c}
481	52: 2	Whole	9.29	13.6	11.445 ^{QC:c}
483	52: 3	Lower half	111	252	181.5 ^{QC:c}
482		Upper half	168	90	129 ^{QC:c}

Table B2-40. Tank 241-T-107 Analytical Results: Zirconium (ICP). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion (Cont'd)			µg/g	µg/g	µg/g
436	Core 51 composite	Whole	85.6	66.3	75.95
495	Core 52 composite	Whole	214	166	190
500		Whole	127	93.8	110.4 ^{QC:c}
Solids: water digest			µg/g	µg/g	µg/g
534	Core 51 composite	Whole	2.1	3.9	3 ^{QC:c}
495	Core 52 composite	Whole	12.2	7.09	9.65 ^{QC:c}
Liquids			µg/mL	µg/mL	µg/mL
393	Core 50 composite	Drainable liquid	0.134	0.134	0.134
442	Core 51 composite	Drainable liquid	0.656	0.641	0.6485
494	Core 52 composite	Drainable liquid	1.76	1.63	1.695

Table B2-41. Tank 241-T-107 Analytical Results: Hexavalent Chromium (Spectrophotometry).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: water digest			µg/g	µg/g	µg/g
534-7769	Core 51 composite	Whole	< 18.5	< 18.4	< 18.45
528-7769	Core 52 composite	Whole	< 19.2	< 19.5	< 19.35

Table B2-42A. Tank 241-T-107 Analytical Results: Total Uranium (LF).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion			µg/g	µg/g	µg/g
257-5740	51: 4	Lower half	7,160	6,940	7,050 ^{QC:c}
294-5740		Lower half	9,020	9,020	9,020
294-5740		Lower half	7,980	7,680	7,830 ^{QC:d}
436-5740	Core 51 composite	Whole	11,400	29,200	20,300 ^{QC:c}
436-6740		Whole	32,900	31,600	32,250 ^{QC:a}
500-5740	Core 52 composite	Whole	16,400	18,300	17,300
500-6740		Whole	18,200	19,000	18,600 ^{QC:d}
500-6740		Whole	18,700	22,900	20,800 ^{QC:d}
Liquids			µg/mL	µg/mL	µg/mL
393-5740	Core 50 composite	Drainable liquid	94.4	96.1	95.25
442-5740	Core 51 composite	Drainable liquid	610	566	588
494-5740	Core 52 composite	Drainable liquid	39.7	41.5	40.6

Table B2-42B. Tank 241-T-107 Analytical Results: Uranium-238.¹

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion			µCi/g	µCi/g	µCi/g
294-5740	51:4	Lower half	0.00303	0.00303	0.00303
436-6740	Core 51 composite	Whole	0.01	0.01	0.01
500-6740	Core 52 composite	Whole	0.00612	0.00638	0.00625
500-6740		Whole	0.00628	0.00769	0.006985
Liquids			µCi/mL	µCi/mL	µCi/mL
393-5781	Core 50 composite	Drainable liquid	3.17E-05	3.23E-05	3.20E-05
442-5781	Core 51 composite	Drainable liquid	2.05E-04	1.90E-04	1.98E-04
494-5781	Core 52 composite	Drainable liquid	1.33E-05	1.39E-05	1.36E-05

Note:

¹Uranium-238 values were calculated from the total uranium (LF) results by assuming the isotopic abundance was equivalent to that found in nature. Uranium concentrations were multiplied by the specific activity of 3.36E-07 Ci/g to obtain ²³⁸U results (Svancara and Pool 1993).

Table B2-43. Tank 241-T-107 Analytical Results: Chloride (IC).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: water digest			µg/g	µg/g	µg/g
534-7771	Core 51 composite	Whole	732	632	682
528-7771	Core 52 composite	Whole	389	409	399
528-7771		Whole	428	420	424
Liquids			µg/mL	µg/mL	µg/mL
393-5771	Core 50 composite	Drainable liquid	199	193	196
442-5771	Core 51 composite	Drainable liquid	1,340	1,340	1,340
494-5771	Core 52 composite	Drainable liquid	851	869	860

Table B2-44. Tank 241-T-107 Analytical Results: Fluoride (IC).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: water digest			µg/g	µg/g	µg/g
534-7771	Core 51 composite	Whole	8,530	9,920	9,225
528-7771	Core 52 composite	Whole	14,000	13,200	13,600 ^{QC:c}
528-7771		Whole	14,300	13,200	13,750 ^{QC:c}
Liquids			µg/mL	µg/mL	µg/mL
393-5771	Core 50 composite	Drainable liquid	170	177	173.5
442-5771	Core 51 composite	Drainable liquid	824	826	825
494-5771	Core 52 composite	Drainable liquid	668	678	673

Table B2-45. Tank 241-T-107 Analytical Results: Nitrate (IC).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: water digest			µg/g	µg/g	µg/g
534-7771	Core 51 composite	Whole	96,400	89,200	92,800
528-7771	Core 52 composite	Whole	54,800	57,700	56,250
528-7771		Whole	58,200	61,200	59,700 ^{QC,a}
Liquids			µg/mL	µg/mL	µg/mL
393-5771	Core 50 composite	Drainable liquid	21,200	21,200	21,200
442-5771	Core 51 composite	Drainable liquid	1.340E+05	1.350E+05	1.345E+05
494-5771	Core 52 composite	Drainable liquid	1.000E+05	1.000E+05	1.000E+05

Table B2-46. Tank 241-T-107 Analytical Results: Nitrite (IC).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: water digest			µg/g	µg/g	µg/g
534-7771	Core 51 composite	Whole	15,700	14,900	15,300
528-7771	Core 52 composite	Whole	7,980	8,290	8,135
528-7771		Whole	8,420	8,730	8,575
Liquids			µg/mL	µg/mL	µg/mL
393-5771	Core 50 composite	Drainable liquid	2,590	2,570	2,580
442-5771	Core 51 composite	Drainable liquid	27,500	27,800	27,650
494-5771	Core 52 composite	Drainable liquid	7,990	8,120	8,055

Table B2-47. Tank 241-T-107 Analytical Results: Phosphate (IC).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: water digest			µg/g	µg/g	µg/g
534-7771	Core 51 composite	Whole	87,000	1.020E+05	94,500
528-7771	Core 52 composite	Whole	1.400E+05	1.250E+05	1.325E+05 ^{OC:c}
528-7771		Whole	1.410E+05	1.280E+05	1.345E+05 ^{OC:c}
Liquids			µg/mL	µg/mL	µg/mL
393-5771	Core 50 composite	Drainable liquid	2,300	2,500	2,400
442-5771	Core 51 composite	Drainable liquid	6,290	6,190	6,240
494-5771	Core 52 composite	Drainable liquid	7,650	7,610	7,630

Table B2-48. Tank 241-T-107 Analytical Results: Sulfate (IC).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: water digest			µg/g	µg/g	µg/g
534-7771	Core 51 composite	Whole	13,000	12,300	12,650
528-7771	Core 52 composite	Whole	6,980	7,260	7,120
528-7771		Whole	7,330	7,620	7,475
Liquids			µg/mL	µg/mL	µg/mL
393-5771	Core 50 composite	Drainable liquid	4,640	4,650	4,645
442-5771	Core 51 composite	Drainable liquid	16,800	16,800	16,800
494-5771	Core 52 composite	Drainable liquid	9,580	9,580	9,580

Table B2-49. Tank 241-T-107 Analytical Results: Ammonia (Distillation).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: water digest			µg/g	µg/g	µg/g
534-7728	Core 51 composite	Whole	< 820	< 816	< 818
528-7728	Core 52 composite	Whole	< 4,200	< 4,260	< 4,230
Liquids			µg/mL	µg/mL	µg/mL
393-5728	Core 50 composite	Drainable liquid	41.6	43	42.3
442-5728	Core 51 composite	Drainable liquid	83.1	80.3	81.7
494-5728	Core 52 composite	Drainable liquid	217	225	221

Table B2-50. Tank 241-T-107 Analytical Results: Nitrite (Spectrophotometry).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: water digest			µg/g	µg/g	µg/g
534-7779	Core 51 composite	Whole	15,000	13,500	14,250
528-7779	Core 52 composite	Whole	7,710	8,240	7,975
Liquids			µg/mL	µg/mL	µg/mL
393-5779	Core 50 composite	Drainable liquid	2,830	2,630	2,730
442-5779	Core 51 composite	Drainable liquid	13,100	12,000	12,550
494-5779	Core 52 composite	Drainable liquid	7,570	7,270	7,420

Table B2-51. Tank 241-T-107 Analytical Results: Cyanide (Spectrometry).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids			$\mu\text{g/g}$	$\mu\text{g/g}$	$\mu\text{g/g}$
310-5777	50: 1R	Subsegment	44.3	52.6	48.45
311-5777	50: 2	Whole	65.6	62.5	64.05
122-5777	50: 3	Whole	39.2	46.2	42.7
409-5777	51: 2	Whole	95.7	94.8	95.25
233-5777	51: 3	Lower half	103	102	102.5
232-5777		Upper half	110	109	109.5
235-5777	51: 4	Lower half	57.4	57.2	57.3
234-5777		Upper half	94.2	88.8	91.5
455-5777	52: 1	Whole	31.5	30.5	31
476-5777	52: 2	Whole	63.4	60	61.7
478-5777	52: 3	Lower half	41.4	45.7	43.55
477-5777		Upper half	50.5	53.7	52.1
427-5777	Core 51 composite	Whole	94.9	96.7	95.8
495-5777	Core 52 composite	Whole	51.4	61.3	56.35
Solids: water digest			$\mu\text{g/g}$	$\mu\text{g/g}$	$\mu\text{g/g}$
534-7778	Core 51 composite	Whole	92.9	90.6	91.75
528-7778	Core 52 composite	Whole	44.9	46.9	45.9
Liquids			$\mu\text{g/mL}$	$\mu\text{g/mL}$	$\mu\text{g/mL}$
393-5778	Core 50 composite	Drainable liquid	13.5	13.4	13.45
442-5778	Core 51 composite	Drainable liquid	152	152	152
494-5778	Core 52 composite	Drainable liquid	40	39.6	39.8

Table B2-52A. Tank 241-T-107 Analytical Results: Semivolatile Organic Analysis Detected Results. (2 sheets)

Tentatively Identified Compound¹	Result²	Duplicate³	Mean
Liquids: Core 50 drainable liquid (93-07218-E1)	µg/mL	µg/mL	µg/mL
dodecane	6	12	9
tetradecane	15	32	24
tributylphosphate	n/d	3	3
tridecane	20	41	31
Solids: Core 52 composite (93-07230-E1)	µg/g	µg/g	µg/g
decamethylcyclopentasiloxane	36	n/d	36
2,2-dimethyldecane	73	110	92
2,2-dimethylheptane	99	140	120
3,3-dimethylhexane ³	96	n/d	96
3,3-dimethylhexane ⁴	8	n/d	8
3,3-dimethylpentane	n/d	44	44
dodecamethylcyclohexasiloxane	110	120	120
3-ethyl-5-methylheptane	10	n/d	10
6-ethyl-2-methyloctane	11 ⁵	140 ⁶	76
hexyl pentyl ether	n/d	12	12
3-methyl-5-propylnonane	n/d	16	16
7-methyltridecane	n/d	19	19
pentadecane	19	37	28
tridecane	22	42	32
2,2,4-trimethyldecane	n/d	17	17
2,2,8-trimethyldecane	16	n/d	16
2,5,6-trimethyldecane	55	80	68
2,2,4-trimethylheptane ⁷	n/d	280	280
2,2,4-trimethylheptane ⁸	n/d	140	140
2,2,6-trimethyloctane	120	n/d	120
2,2,6-trimethyloctane	31	n/d	31

Table B2-52A. Tank 241-T-107 Analytical Results: Semivolatile Organic Analysis Detected Results. (2 sheets)

Tentatively Identified Compound ¹	Result ²	Duplicate ³	Mean
Solids: Core 52 composite (93-07230-E1) (Cont'd)	µg/g	µg/g	µg/g
2,3,7-trimethyloctane ⁹	53	77	65
2,3,7-trimethyloctane ¹⁰	16	23	20
2,4,6-trimethyloctane	200	n/d	200

Notes:

n/d = not detected. No estimate of detection limit was available.

¹C compound identification is based on best match of tentatively identified compound's mass spectrum to spectral library mass spectra. The tentatively identified compounds may or may not actually be present in the original tank waste.

²Tentatively identified compound concentrations are estimated using the response factor of the internal standard with the retention time nearest that of the tentatively identified compound.

³Identified at retention time of 12.0 minutes

⁴Identified at retention time of 12.7 minutes

⁵Identified at retention time of 12.4 minutes; this retention time more closely matches that of 3-methyl-5-propylnonane (12.4 minutes) in the duplicate

⁶Identified at retention time of 12.0 minutes; this retention time more closely matches that of 3,3-dimethylhexane (12.0 minutes) in the sample

⁷Identified at retention time of 9.8 minutes

⁸Identified at retention time of 10.3 minutes

⁹Identified at retention time of 10.8 minutes

¹⁰Identified at retention time of 11.5 minutes

Table B2-52B. Tank 241-T-107 Analytical Results: Semivolatile Organic Analysis
Nondetected Results.¹ (5 sheets)

Analyte	µg/mL
Liquid Core composite	
1,2,4-Trichlorobenzene	< 5.00
1,2-Dichlorobenzene	< 5.00
1,3-Dichlorobenzene	< 5.00
1,4-Dichlorobenzene	< 5.00
2,4,5-Trichlorophenol	< 25.0
2,4,6-Trichlorophenol	< 5.00
2,4-Dichlorophenol	< 5.00
2,4-Dimethylphenol	< 5.00
2,4-Dinitrophenol	< 25.0
2,4-Dinitrotoluene	< 5.00
2,6-Dinitrotoluene	< 5.00
2-Chloronaphthalene	< 5.00
2-Chlorophenol	< 5.00
2-Methylnaphthalene	< 5.00
2-Methylphenol	< 5.00
2-Nitroaniline	< 25.0
2-Nitrophenol	< 5.00
3,3-Dichlorobenzidine	< 10.0
3-Nitroaniline	< 25.0
4,6-Dinitro-o-cresol	< 25.0
4-Bromophenylphenyl ether	< 5.00
4-Chloro-3-methylphenol	< 5.00
4-Chloroaniline	< 5.00
4-Chlorophenylphenyl ether	< 5.00
4-Methylphenol	< 5.00
4-Nitroaniline	< 25.0
4-Nitrophenol	< 25.0
Acenaphthene	< 5.00
Acenaphthylene	< 5.00

Table B2-52B. Tank 241-T-107 Analytical Results: Semivolatile Organic Analysis
Nondetected Results.¹ (5 sheets)

Analyte	µg/mL
Liquid Core composite (Cont'd)	
Anthracene	< 5.00
Benzo(a)anthracene	< 5.00
Benzo(a)pyrene	< 5.00
Benzo(b)fluoranthene	< 5.00
Benzo(ghi)perylene	< 5.00
Benzo(k)fluoranthene	< 5.00
Benzoic acid	< 25.0
Benzyl alcohol	< 5.00
Bis(2-Chloroethoxy)methane	< 5.00
Bis(2-chloroethyl) ether	< 5.00
Bis(2-Chloroisopropyl) ether	< 5.00
Bis(2-ethylhexyl)phthalate	< 5.00
Butylbenzylphthalate	< 5.00
Chrysene	< 5.00
Di-n-butylphthalate	< 5.00
Di-n-octylphthalate	< 5.00
Dibenz[a, h]anthracene	< 5.00
Dibenzofuran	< 5.00
Diethylphthalate	< 5.00
Dimethyl phthalate	< 5.00
Fluoranthene	< 5.00
Fluorene	< 5.00
Hexachlorobenzene	< 5.00
Hexachlorobutadiene	< 5.00
Hexachlorocyclopentadiene	< 5.00
Hexachloroethane	< 5.00
Indeno(1,2,3-cd)pyrene	< 5.00
Isophorone	< 5.00
Nitrobenzene	< 5.00

Table B2-52B. Tank 241-T-107 Analytical Results: Semivolatile Organic Analysis
Nondetected Results.¹ (5 sheets)

Analyte	µg/mL
Liquid Core Composite (Cont'd)	
Pentachlorophenol	<25.0
Phenanthrene	<5.00
Phenol	<5.00
Pyrene	<5.00
Analyte	µg/g
Solid Core Composite	
1,2,4-Trichlorobenzene	<18.5
1,2-Dichlorobenzene	<18.5
1,3-Dichlorobenzene	<18.5
1,4-Dichlorobenzene	<18.5
2,4,5-Trichlorophenol	<90.5
2,4,6-Trichlorophenol	<18.5
2,4-Dichlorophenol	<18.5
2,4-Dimethylphenol	<18.5
2,4-Dinitrophenol	<90.5
2,4-Dinitrotoluene	<18.5
2,6-Dinitrotoluene	<18.5
2-Chloronaphthalene	<18.5
2-Chlorophenol	<18.5
2-Methylnaphthalene	<18.5
2-Methylphenol	<18.5
2-Nitroaniline	<90.5
2-Nitrophenol	<18.5
3,3-Dichlorobenzidine	<36.0
3-Nitroaniline	<90.5
4,6-Dinitro-o-cresol	<90.5
4-Bromophenylphenyl ether	<18.5
4-Chloro-3-methylphenol	<18.5
4-Chloroaniline	<18.5

Table B2-52B. Tank 241-T-107 Analytical Results: Semivolatile Organic Analysis
Nondetected Results.¹ (5 sheets)

Analyte	µg/g
Solid Core Composite (Cont'd)	
4-Chlorophenylphenyl ether	< 18.5
4-Methylphenol	< 18.5
4-Nitroaniline	< 90.5
4-Nitrophenol	< 90.5
Acenaphthene	< 18.5
Acenaphthylene	< 18.5
Anthracene	< 18.5
Benzo(a)anthracene	< 18.5
Benzo(a)pyrene	< 18.5
Benzo(b)fluoranthene	< 18.5
Benzo(ghi)perylene	< 18.5
Benzo(k)fluoranthene	< 18.5
Benzoic acid	< 90.5
Benzyl alcohol	< 18.5
Bis(2-Chloroethoxy)methane	< 18.5
Bis(2-chloroethyl) ether	< 18.5
Bis(2-Chloroisopropyl) ether	< 18.5
Bis(2-ethylhexyl)phthalate	< 18.5
Butylbenzylphthalate	< 18.5
Chrysene	< 18.5
Di-n-butylphthalate	< 18.5
Di-n-octylphthalate	< 18.5
Dibenz[a,h]anthracene	< 18.5
Dibenzofuran	< 18.5
Diethylphthalate	< 18.5
Dimethyl phthalate	< 18.5
Fluoranthene	< 18.5
Fluorene	< 18.5
Hexachlorobenzene	< 18.5

Table B2-52B. Tank 241-T-107 Analytical Results: Semivolatile Organic Analysis
Nondetected Results.¹ (5 sheets)

Analyte	µg/g
Solid Core Composite (Cont'd)	
Hexachlorobutadiene	< 18.5
Hexachlorocyclopentadiene	< 18.5
Hexachloroethane	< 18.5
Indeno(1,2,3-cd)pyrene	< 18.5
Isophorone	< 18.5
Nitrobenzene	< 18.5
Pentachlorophenol	< 90.5
Phenanthrene	< 18.5
Phenol	< 18.5
Pyrene	< 18.5

Note:

¹The detection limit listed for each analyte is the contract required quantitation limit not the method detection limit.

Table B2-53. Tank 241-T-107 Analytical Results: Total Carbon (Persulfate Oxidation).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Triplicate	Mean
Solids			$\mu\text{g/g}$	$\mu\text{g/g}$	$\mu\text{g/g}$	$\mu\text{g/g}$
93-07215-J1	50: 1R	Subsegment	2,270	2,250	---	2,260
93-07216-J1	50: 2	Whole	3,540	3,840	---	3,690
93-07219-J1	51: 2	Whole	4,750	5,470	---	5,110
93-07221-J1	51: 3	Lower half	3,510	3,550	---	3,530
93-07220-J1		Upper half	4,690	4,140	---	4,415
93-07223-J1	51: 4	Lower half	1,720	2,140	---	1,930 ^{QC:c}
93-07222-J1		Upper half	3,150	2,950	---	3,050
93-07226-J1	52: 1	Whole	7,070	4,350	3,810	5,710 ^{QC:c}
93-07227-J1	52: 2	Whole	4,290	3,560	---	3,925
93-07229-J1	52: 3	Lower half	1,680	1,830	---	1,755
93-07228-J1		Upper half	2,350	1,720	---	2,035 ^{QC:c}
93-07225-J1	Core 51 composite	Whole	2,690	2,270	---	2,480
93-07230-J1	Core 52 composite	Whole	1,520	1,760	---	1,640

Table B2-54A. Tank 241-T-107 Analytical Results: Total Organic Carbon (Persulfate Oxidation).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Triplicate	Mean
Solids			$\mu\text{g/g}$	$\mu\text{g/g}$	$\mu\text{g/g}$	$\mu\text{g/g}$
93-07215-J1	50: 1R	Subsegment	510	500	---	505
93-07216-J1	50: 2	Whole	600	710	---	655
93-07219-J1	51: 2	Whole	960	1,230	---	1,095 ^{QC:c}
93-07221-J1	51: 3	Lower half	810	1,000	---	905
93-07220-J1		Upper half	1,330	1,200	---	1,265
93-07223-J1	51: 4	Lower half	260	280	---	270
93-07222-J1		Upper half	290	240	---	265
93-07226-J1	52: 1	Whole	3,630	2,000	1,900	2,815 ^{QC:c}
93-07227-J1	52: 2	Whole	1,030	910	---	970
93-07229-J1	52: 3	Lower half	220	310	---	265 ^{QC:c}
93-07228-J1		Upper half	720	650	---	685
93-07225-J1	Core 51 composite	Whole	390	410	---	400
93-07230-J1	Core 52 composite	Whole	370	270	---	320 ^{QC:c}

Table B2-54B. Tank 241-T-107 Analytical Results: Total Organic Carbon (Furnace Oxidation).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: water digest			$\mu\text{g/g}$	$\mu\text{g/g}$	$\mu\text{g/g}$
534-7726	Core 51 composite	Whole	1,520	1,350	1,435
528-7726	Core 52 composite	Whole	2,000	1,920	1,960
Liquids			$\mu\text{g/mL}$	$\mu\text{g/mL}$	$\mu\text{g/mL}$
393-5726	Core 50 composite	Drainable liquid	1,100	1,200	1,150
442-5726	Core 51 composite	Drainable liquid	1,050	1,080	1,065
494-5726	Core 52 composite	Drainable liquid	340	369	354.5

Table B2-55A. Tank 241-T-107 Analytical Results: Total Inorganic Carbon (Persulfate Oxidation).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Triplicate	Mean
Solids			µg/g	µg/g		µg/g
93-07215-J1	50: 1R	Subsegment	1,760	1,750	---	1,760
93-07216-J1	50: 2	Whole	2,940	3,130	---	3,040
93-07219-J1	51: 2	Whole	3,800	4,240	---	4,020
93-07221-J1	51: 3	Lower half	2,700	2,550	---	2,630
93-07220-J1		Upper half	3,360	2,940	---	3,150
93-07223-J1	51: 4	Lower half	1,470	1,860	---	1,670 ^{QC:c}
93-07222-J1		Upper half	2,860	2,700	---	2,780
93-07226-J1	52: 1	Whole	3,440	2,350	1,920	2,570 ^{QC:c}
93-07227-J1	52: 2	Whole	3,260	2,650	---	2,960 ^{QC:c}
93-07229-J1	52: 3	Lower half	1,460	1,520	---	1,490
93-07228-J1		Upper half	1,630	1,070	---	1,350 ^{QC:c}
93-07225-J1	Core 51 composite	Whole	2,300	1,860	---	2,080 ^{QC:c}
93-07230-J1	Core 52 composite	Whole	1,150	1,490	---	1,320 ^{QC:c}

Table B2-55B. Tank 241-T-107 Analytical Results: Total Inorganic Carbon (Furnace Oxidation).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: water digest			µg/g	µg/g	µg/g
534-7727	Core 51 composite	Whole	5,640	5,710	5,675
528-7727	Core 52 composite	Whole	2,990	2,560	2,775
Liquids			µg/mL	µg/mL	µg/mL
393-5727	Core 50 composite	Drainable liquid	516	508	512
442-5727	Core 51 composite	Drainable liquid	4,580	4,510	4,545
494-5727	Core 52 composite	Drainable liquid	346	332	339

Table B2-56A. Tank 241-T-107 Analytical Results: Total Alpha
(Alpha Proportional Counting).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion			$\mu\text{Ci/g}$	$\mu\text{Ci/g}$	$\mu\text{Ci/g}$
436-6725	Core 51 composite	Whole	0.475	0.471	0.473
500-6725	Core 52 composite	Whole	0.379	0.411	0.395
Solids: water digest			$\mu\text{Ci/g}$	$\mu\text{Ci/g}$	$\mu\text{Ci/g}$
430-7725	Core 51 composite	Whole	5.440E-04	4.960E-04	5.200E-04
528-7725	Core 52 composite	Whole	0.0055	0.00426	0.00488 ^{QC:c}
528-7725		Whole	0.0116	0.00839	0.009995 ^{QC:c}
Liquids			$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$
393-5725	Core 50 composite	Drainable liquid	9.740E-04	8.540E-04	9.140E-04
442-5725	Core 51 composite	Drainable liquid	0.0171	0.0161	0.0166
494-5725	Core 52 composite	Drainable liquid	5.420E-04	4.800E-04	5.110E-04

Table B2-56B. Tank 241-T-107 Analytical Results: Total Alpha Pu
(Alpha Proportional Counting).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion			$\mu\text{Ci/g}$	$\mu\text{Ci/g}$	$\mu\text{Ci/g}$
93-7224-H-1	51: 4	Lower half	0.208	0.264	0.236 ^{QC:c}
93-7230-H-1	Core 52 composite	Whole	0.190	0.183	0.187

Table B2-57. Tank 241-T-107 Analytical Results: Total Beta
(Beta Proportional Counting).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion			$\mu\text{Ci/g}$	$\mu\text{Ci/g}$	$\mu\text{Ci/g}$
436-6725	Core 51 composite	Whole	392	415	403.5
500-6725	Core 52 composite	Whole	240	274	257
Solids: water digest			$\mu\text{Ci/g}$	$\mu\text{Ci/g}$	$\mu\text{Ci/g}$
430-7725	Core 51 composite	Whole	16.1	16.9	16.5
528-7725	Core 52 composite	Whole	11	9.68	10.34
Liquids			$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$
393-5725	Core 50 composite	Drainable liquid	2.54	2.45	2.495
442-5725	Core 51 composite	Drainable liquid	25.7	24.7	25.2
494-5725	Core 52 composite	Drainable liquid	7.5	7.61	7.555

Table B2-58. Tank 241-T-107 Analytical Results: Strontium-90
(Beta Proportional Counting). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion			$\mu\text{Ci/g}$	$\mu\text{Ci/g}$	$\mu\text{Ci/g}$
387-6786	50: 1R	Subsegment	33.7	29.6	31.65
309-6786	50: 2	Whole	155	150	152.5
322-6786	50: 3	Whole	126	125	125.5
406-6786	51: 2	Whole	188	192	190
255-6786	51: 3	Lower half	244	240	242
254-6786		Upper half	212	190	201
257-6786	51: 4	Lower half	26.9	28	27.45
256-6786		Upper half	221	232	226.5
455-6786	52: 1	Whole	179	151	165 ^{QC:c}
481-6786	52: 2	Whole	92.2	83.7	87.95
483-6786	52: 3	Lower half	21.4	16.2	18.8 ^{QC:c}
483-6786		Lower half	19.6	15.1	17.35 ^{QC:c}
482-6786		Upper half	92.5	99.2	95.85
436-6786	Core 51 composite	Whole	132	131	131.5
500-6786	Core 52 composite	Whole	92	78.9	85.45

Table B2-58. Tank 241-T-107 Analytical Results: Strontium-90
(Beta Proportional Counting). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Liquids			$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$
393-5786	Core 50 composite	Drainable liquid	0.0105	0.0111	0.0108
442-5786	Core 51 composite	Drainable liquid	0.118	0.127	0.123
494-5786	Core 52 composite	Drainable liquid	0.0455	0.0443	0.0449

Table B2-59. Tank 241-T-107 Analytical Results: Americium-241 (Alpha Spectrometry).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion			$\mu\text{Ci/g}$	$\mu\text{Ci/g}$	$\mu\text{Ci/g}$
436-6781	Core 51 composite	Whole	0.0115	0.0111	0.0113 ^{QC:c}
500-6781	Core 52 composite	Whole	0.0161	0.0175	0.0168
500-6781		Whole	0.0146	0.0189	0.01675 ^{QC:c,c}
Liquids			$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$
393-5781	Core 50 composite	Drainable liquid	< 4.030E-05	< 2.670E-05	< 3.350E-05 ^{QC:c}
442-5781	Core 51 composite	Drainable liquid	2.040E-04	< 5.08E-05	1.27E-04 ^{QC:c}
494-5781	Core 52 composite	Drainable liquid	6.790E-05	7.220E-05	7.005E-05 ^{QC:c}

Table B2-60. Tank 241-T-107 Analytical Results: Plutonium-238 (Alpha Spectrometry).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion			μCi/g	μCi/g	μCi/g
294-6781	51: 4	Lower half	<0.017	<0.0134	<0.0152 ^{QC:c,e}
436-6781	Core 51 composite	Whole	---	<0.00718	<0.00718 ^{QC:c}
436-6781		Whole	<0.0113	<0.0127	<0.0120 ^{QC:c}
436-6781		Whole	<0.00613	0.0166	<0.011365 ^{QC:c,e}
500-6781	Core 52 composite	Whole	<0.0161	<0.0165	<0.0163 ^{QC:c}
500-6781		Whole	<0.00635	<0.00587	<0.00611 ^{QC:c}
Liquids			μCi/mL	μCi/mL	μCi/mL
393-5781	Core 50 composite	Drainable liquid	<9.010E-05	<9.010E-05	<9.010E-05
442-5781	Core 51 composite	Drainable liquid	0.00244	0.00256	0.0025 ^{QC:c}
494-5781	Core 52 composite	Drainable liquid	<1.710E-04	<1.140E-04	<1.425E-04 ^{QC:c}

Table B2-61. Tank 241-T-107 Analytical Results: Plutonium-239/40 (Alpha Spectrometry).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion			μCi/g	μCi/g	μCi/g
294-6781	51: 4	Lower half	0.188	0.157	0.1725 ^{QC:c}
436-6781	Core 51 composite	Whole	---	0.124	0.124 ^{QC:c}
436-6781		Whole	0.131	0.108	0.1195 ^{QC:b,c}
436-6781		Whole	0.129	0.0925	0.11075 ^{QC:c,e}
500-6781	Core 52 composite	Whole	0.153	0.184	0.1685 ^{QC:c}
500-6781		Whole	0.176	0.220	0.198 ^{QC:c,e}
Liquids			μCi/mL	μCi/mL	μCi/mL
393-5781	Core 50 composite	Drainable liquid	<6.570E-05	<6.600E-05	<6.585E-05
442-5781	Core 51 composite	Drainable liquid	0.0111	0.0117	0.0114 ^{QC:c}
494-5781	Core 52 composite	Drainable liquid	<6.700E-05	<6.280E-05	<6.490E-05 ^{QC:c}

Table B2-62. Tank 241-T-107 Analytical Results: Plutonium-238 to Pu Ratio (Mass Spectrometry).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion			%	%	%
93-7224-H-2	51: 4	Lower half	0.003	0.005	0.004 ^{QC:c}
93-7230-H-1	Core 52 composite	Whole	0.006	0.008	0.007 ^{QC:c}

Table B2-63. Tank 241-T-107 Analytical Results: Plutonium-239 to Pu Ratio (Mass Spectrometry).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion			%	%	%
93-7224-H-2	51: 4	Lower half	98.101	98.115	98.108
93-7230-H-1	Core 52 composite	Whole	98.133	98.011	98.072

Table B2-64. Tank 241-T-107 Analytical Results: Plutonium-240 to Pu Ratio (Mass Spectrometry).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion			%	%	%
93-7224-H-2	51: 4	Lower half	1.871	1.85	1.8605
93-7230-H-1	Core 52 composite	Whole	1.833	1.94	1.8865

Table B2-65. Tank 241-T-107 Analytical Results: Plutonium-241 to Pu Ratio (Mass Spectrometry).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion			%	%	%
93-7224-H-2	51: 4	Lower half	0.02	0.02	0.02
93-7230-H-1	Core 52 composite	Whole	0.02	0.03	0.025 ^{QC:c}

Table B2-66. Tank 241-T-107 Analytical Results: Plutonium-242 to Pu Ratio (Mass Spectrometry).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion			%	%	%
93-7224-H-2	51: 4	Lower half	0.01	0.01	0.01
93-7230-H-1	Core 52 composite	Whole	0.01	0.02	0.015 ^{QC:c}

Table B2-67. Tank 241-T-107 Analytical Results: Uranium-234 to U Ratio (Mass Spectrometry).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion			%	%	%
93-7224-H-2	51: 4	Lower half	0.005	0.005	0.005
93-7230-H-1	Core 52 composite	Whole	0.005	0.005	0.005

Table B2-68. Tank 241-T-107 Analytical Results: Uranium-235 to U Ratio (Mass Spectrometry).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion			%	%	%
93-7224-H-2	51: 4	Lower half	0.688	0.695	0.6915
93-7230-H-1	Core 52 composite	Whole	0.687	0.686	0.6865

Table B2-69. Tank 241-T-107 Analytical Results: Uranium-236 to U Ratio (Mass Spectrometry).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion			%	%	%
93-7224-H-2	51: 4	Lower half	0.004	0.004	0.004
93-7230-H-1	Core 52 composite	Whole	0.005	0.004	0.0045 ^{QC:e}

Table B2-70. Tank 241-T-107 Analytical Results: Uranium-238 to U Ratio (Mass Spectrometry).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion			%	%	%
93-7224-H-2	51: 4	Lower half	99.303	99.296	99.2995
93-7230-H-1	Core 52 composite	Whole	99.303	99.304	99.3035

Table B2-71. Tank 241-T-107 Analytical Results: Americium-241 (GEA).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion			$\mu\text{Ci/g}$	$\mu\text{Ci/g}$	$\mu\text{Ci/g}$
387-6730	50: 1R	Subsegment	<0.0669	<0.0657	<0.0663
309-6730	50: 2	Whole	<0.151	<0.154	<0.1525
322-6730	50: 3	Whole	<0.0842	<0.0881	<0.08615
406-6730	51: 2	Whole	<0.213	<0.216	<0.2145
255-6730	51: 3	Lower half	<0.148	<0.151	<0.1495
254-6730		Upper half	<0.136	<0.126	<0.131
257-6730	51: 4	Lower half	<0.0858	<0.0834	<0.0846
256-6730		Upper half	<0.139	<0.148	<0.1435
455-6730	52: 1	Whole	0.253	<0.0949	<0.17395 ^{0c:c}
481-6730	52: 2	Whole	<0.0528	<0.0498	<0.0513
483-6730	52: 3	Lower half	<0.0361	<0.0358	<0.03595
482-6730		Upper half	<0.0521	<0.0506	<0.05135
436-6782	Core 51 composite	Whole	<0.0593	<0.0614	<0.06035
500-6730	Core 52 composite	Whole	<0.0813	<0.0873	<0.0843
Solids: water digest			$\mu\text{Ci/g}$	$\mu\text{Ci/g}$	$\mu\text{Ci/g}$
534-7730	Core 51 composite	Whole	<0.0281	<0.0267	<0.0274
534-9730		Whole	<0.349	<0.385	<0.367
528-7730	Core 52 composite	Whole	<0.0222	<0.0228	<0.0225
528-9730		Whole	<0.192	<0.192	<0.192
Liquids			$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$
393-5782	Core 50 composite	Drainable liquid	<0.00733	<0.00733	<0.00733
442-5782	Core 51 composite	Drainable liquid	<0.0323	<0.033	<0.03265
494-5782	Core 52 composite	Drainable liquid	<0.00568	<0.0059	<0.00579

Table B2-72. Tank 241-T-107 Analytical Results: Cerium/Praeseodymium-144 (GEA).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion			$\mu\text{Ci/g}$	$\mu\text{Ci/g}$	$\mu\text{Ci/g}$
387-6730	50: 1R	Subsegment	<0.0882	<0.0875	<0.08785
309-6730	50: 2	Whole	<0.202	<0.198	<0.2
322-6730	50: 3	Whole	<0.11	<0.118	<0.114
406-6730	51: 2	Whole	<0.51	<0.513	<0.5115
255-6730	51: 3	Lower half	<0.317	<0.31	<0.3135
254-6730		Upper half	<0.292	<0.271	<0.2815
257-6730	51: 4	Lower half	<0.198	<0.197	<0.1975
256-6730		Upper half	<0.297	<0.316	<0.3065
455-6730	52: 1	Whole	<0.147	<0.133	<0.14
481-6730	52: 2	Whole	<0.319	<0.313	<0.316
483-6730	52: 3	Lower half	<0.284	<0.282	<0.283
482-6730		Upper half	<0.305	<0.314	<0.3095
436-6782	Core 51 composite	Whole	<0.36	<0.359	<0.3595
500-6730	Core 52 composite	Whole	<0.116	<0.12	<0.118
Solids: water digest			$\mu\text{Ci/g}$	$\mu\text{Ci/g}$	$\mu\text{Ci/g}$
534-7730	Core 51 composite	Whole	<0.0409	<0.0395	<0.0402
534-9730		Whole	<0.271	<0.305	<0.288
528-7730	Core 52 composite	Whole	<0.0579	<0.0579	<0.0579
528-9730		Whole	<0.15	<0.158	<0.154
Liquids			$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$
393-5782	Core 50 composite	Drainable liquid	<0.00999	<0.0101	<0.010045
442-5782	Core 51 composite	Drainable liquid	<0.0478	<0.0488	<0.0483
494-5782	Core 52 composite	Drainable liquid	<0.00834	<0.00861	<0.008475

Table B2-73. Tank 241-T-107 Analytical Results: Cesium-134 (GEA).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion			μCi/g	μCi/g	μCi/g
387-6730	50: 1R	Subsegment	<0.0061	<0.00563	<0.005865
309-6730	50: 2	Whole	<0.0133	<0.0143	<0.0138
322-6730	50: 3	Whole	<0.00688	<0.00673	<0.006805
406-6730	51: 2	Whole	<0.0139	<0.0151	<0.0145
255-6730	51: 3	Lower half	<0.0114	<0.0102	<0.0108
254-6730		Upper half	<0.00879	<0.00919	<0.00899
257-6730	51: 4	Lower half	<0.00849	<0.00906	<0.008775
256-6730		Upper half	<0.0105	<0.01	<0.01025
455-6730	52: 1	Whole	<0.009	<0.00814	<0.00857
481-6730	52: 2	Whole	<0.0246	<0.0225	<0.02355
483-6730	52: 3	Lower half	<0.0219	<0.0254	<0.02365
482-6730		Upper half	<0.0209	<0.024	<0.02245
436-6782	Core 51 composite	Whole	<0.0217	<0.0221	<0.0219
500-6730	Core 52 composite	Whole	<0.0055	<0.00584	<0.00567
Solids: water digest			μCi/g	μCi/g	μCi/g
534-7730	Core 51 composite	Whole	<0.00127	<0.00129	<0.00128
534-9730		Whole	<0.0121	<0.0139	<0.013
528-7730	Core 52 composite	Whole	<0.00204	<0.00164	<0.00184 ^{QC:c}
528-9730		Whole	<0.00766	<0.0075	<0.00758
Liquids			μCi/mL	μCi/mL	μCi/mL
393-5782	Core 50 composite	Drainable liquid	<4.660E-04	<5.140E-04	<4.900E-04
442-5782	Core 51 composite	Drainable liquid	<0.0014	<0.00135	<0.001375
494-5782	Core 52 composite	Drainable liquid	<2.660E-04	<2.720E-04	<2.690E-04

Table B2-74. Tank 241-T-107 Analytical Results: Cesium-137 (GEA).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion			$\mu\text{Ci/g}$	$\mu\text{Ci/g}$	$\mu\text{Ci/g}$
387-6730	50: 1R	Subsegment	7.08	6.97	7.025
309-6730	50: 2	Whole	12.1	11.5	11.8
322-6730	50: 3	Whole	5.65	6.43	6.04
406-6730	51: 2	Whole	97.7	102	99.85
406-6730		Whole	96.2	96.7	96.45
255-6730	51: 3	Lower half	17.1	17	17.05
254-6730		Upper half	16.2	14.4	15.3
257-6730	51: 4	Lower half	13.7	13.5	13.6
256-6730		Upper half	16.7	19	17.85
455-6730	52: 1	Whole	10.8	10.9	10.85
481-6730	52: 2	Whole	10.9	9.66	10.28
483-6730	52: 3	Lower half	10.6	10.7	10.65
482-6730		Upper half	7.39	8.26	7.825
436-6782	Core 51 composite	Whole	13.5	14.2	13.85
500-6730	Core 52 composite	Whole	10.3	11	10.65
Solids: water digest			$\mu\text{Ci/g}$	$\mu\text{Ci/g}$	$\mu\text{Ci/g}$
534-7730	Core 51 composite	Whole	12.4	11.6	12
534-9730		Whole	17.1	17.7	17.4
528-7730	Core 52 composite	Whole	6.39	6.59	6.49
528-9730		Whole	5.67	7.43	6.55 ^{QC:c}
Liquids			$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$
393-5782	Core 50 composite	Drainable liquid	1.7	1.73	1.715
442-5782	Core 51 composite	Drainable liquid	18	18.7	18.35
494-5782	Core 52 composite	Drainable liquid	5.06	5.4	5.23

Table B2-75. Tank 241-T-107 Analytical Results: Cobalt-60 (GEA).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion			$\mu\text{Ci/g}$	$\mu\text{Ci/g}$	$\mu\text{Ci/g}$
387-6730	50: 1R	Subsegment	<0.00624	<0.00687	<0.006555
309-6730	50: 2	Whole	<0.0114	<0.0127	<0.01205
322-6730	50: 3	Whole	<0.00515	<0.00594	<0.005545
406-6730	51: 2	Whole	<0.0071	<0.0105	<0.0088 ^{QC:c}
255-6730	51: 3	Lower half	0.0376	<0.008	<0.0228 ^{QC:c}
254-6730		Upper half	<0.00893	<0.00878	<0.008855
257-6730	51: 4	Lower half	<0.00892	<0.00772	<0.00832
256-6730		Upper half	<0.00889	0.0264	<0.017645 ^{QC:c}
455-6730	52: 1	Whole	<0.00639	<0.00639	<0.00639
481-6730	52: 2	Whole	<0.0251	<0.0207	<0.0229
483-6730	52: 3	Lower half	<0.022	<0.0254	<0.0237
482-6730		Upper half	<0.0213	<0.019	<0.02015
436-6782	Core 51 composite	Whole	<0.0239	<0.0225	<0.0232
500-6730	Core 52 composite	Whole	<0.00496	<0.00658	<0.00577 ^{QC:c}
Solids: water digest			$\mu\text{Ci/g}$	$\mu\text{Ci/g}$	$\mu\text{Ci/g}$
534-7730	Core 51 composite	Whole	<0.0012	<0.00145	<0.001325
534-9730		Whole	0.0284	0.0296	0.029
528-7730	Core 52 composite	Whole	<0.00199	<0.00208	<0.002035
528-9730		Whole	0.007	0.009	0.008 ^{QC:c}
Liquids			$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$
393-5782	Core 50 composite	Drainable liquid	<4.060E-04	<5.280E-04	<4.670E-04 ^{QC:c}
442-5782	Core 51 composite	Drainable liquid	<0.00135	<0.0011	<0.001225
494-5782	Core 52 composite	Drainable liquid	<1.240E-04	<1.240E-04	<1.240E-04

Table B2-76. Tank 241-T-107 Analytical Results: Europium-154 (GEA).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion			$\mu\text{Ci/g}$	$\mu\text{Ci/g}$	$\mu\text{Ci/g}$
387-6730	50: 1R	Subsegment	0.108	0.089	0.0985
309-6730	50: 2	Whole	<0.0296	<0.0314	<0.0305
322-6730	50: 3	Whole	<0.0169	<0.0178	<0.01735
406-6730	51: 2	Whole	0.31	0.317	0.3135
255-6730	51: 3	Lower half	0.0437	<0.0259	<0.0348 ^{QC:e}
254-6730		Upper half	<0.0207	<0.0211	<0.0209
257-6730	51: 4	Lower half	<0.0212	<0.0211	<0.02115
256-6730		Upper half	<0.0185	<0.0252	<0.02185 ^{QC:e}
455-6730	52: 1	Whole	1.21	0.944	1.077 ^{QC:e}
481-6730	52: 2	Whole	<0.0648	<0.0737	<0.06925
483-6730	52: 3	Lower half	<0.0676	<0.0642	<0.0659
482-6730		Upper half	<0.0774	<0.0767	<0.07705
436-6782	Core 51 composite	Whole	<0.0598	<0.0644	<0.0621
500-6730	Core 52 composite	Whole	<0.0181	0.0688	<0.04345 ^{QC:e}
Solids: water digest			$\mu\text{Ci/g}$	$\mu\text{Ci/g}$	$\mu\text{Ci/g}$
534-7730	Core 51 composite	Whole	<0.00391	<0.0034	<0.003655
534-9730		Whole	0.12	0.149	0.1345 ^{QC:e}
528-7730	Core 52 composite	Whole	<0.00436	<0.00582	<0.00509 ^{QC:e}
528-9730		Whole	0.125	0.138	0.1315
Liquids			$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$
393-5782	Core 50 composite	Drainable liquid	<0.00131	<0.00144	<0.001375
442-5782	Core 51 composite	Drainable liquid	<0.00423	<0.00399	<0.00411
494-5782	Core 52 composite	Drainable liquid	<5.500E-04	<5.290E-04	<5.395E-04

Table B2-77. Tank 241-T-107 Analytical Results: Europium-155 (GEA).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion			$\mu\text{Ci/g}$	$\mu\text{Ci/g}$	$\mu\text{Ci/g}$
387-6730	50: 1R	Subsegment	0.0892	0.0826	0.0859
309-6730	50: 2	Whole	<0.0689	<0.0701	<0.0695
322-6730	50: 3	Whole	<0.0389	<0.0411	<0.04
406-6730	51: 2	Whole	<0.14	<0.141	<0.1405
255-6730	51: 3	Lower half	<0.0935	<0.0949	<0.0942
254-6730		Upper half	<0.086	<0.0809	<0.08345
257-6730	51: 4	Lower half	<0.0535	<0.0532	<0.05335
256-6730		Upper half	<0.0879	<0.0949	<0.0914
455-6730	52: 1	Whole	1.07	0.769	0.9195 ^{QC:c}
481-6730	52: 2	Whole	<0.0827	<0.0807	<0.0817
483-6730	52: 3	Lower half	<0.0686	<0.0666	<0.0676
482-6730		Upper half	<0.0789	<0.0811	<0.08
436-6782	Core 51 composite	Whole	<0.0886	<0.0921	<0.09035
500-6730	Core 52 composite	Whole	<0.0387	<0.0407	<0.0397
Solids: water digest			$\mu\text{Ci/g}$	$\mu\text{Ci/g}$	$\mu\text{Ci/g}$
534-7730	Core 51 composite	Whole	<0.0125	<0.0123	<0.0124
534-9730		Whole	<0.159	<0.206	<0.1825 ^{QC:c}
528-7730	Core 52 composite	Whole	<0.0149	<0.0151	<0.015
528-9730		Whole	0.107	0.139	0.123 ^{QC:c}
Liquids			$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$
393-5782	Core 50 composite	Drainable liquid	<0.00307	<0.00314	<0.003105
442-5782	Core 51 composite	Drainable liquid	<0.0147	<0.015	<0.01485
494-5782	Core 52 composite	Drainable liquid	<0.00257	<0.00265	<0.00261

Table B2-78. Tank 241-T-107 Analytical Results: Potassium-40 (GEA).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion			$\mu\text{Ci/g}$	$\mu\text{Ci/g}$	$\mu\text{Ci/g}$
387-6730	50: 1R	Subsegment	<0.154	<0.154	<0.154
309-6730	50: 2	Whole	<0.314	<0.312	<0.313
322-6730	50: 3	Whole	<0.155	<0.155	<0.155
406-6730	51: 2	Whole	<0.286	<0.293	<0.2895
255-6730	51: 3	Lower half	<0.282	<0.293	<0.2875
254-6730		Upper half	<0.276	<0.272	<0.274
257-6730	51: 4	Lower half	<0.287	<0.278	<0.2825
256-6730		Upper half	<0.282	<0.283	<0.2825
455-6730	52: 1	Whole	0.17	<0.155	<0.1625
481-6730	52: 2	Whole	<0.717	<0.732	<0.7245
483-6730	52: 3	Lower half	<0.72	<0.72	<0.72
482-6730		Upper half	<0.724	<0.708	<0.716
436-6782	Core 51 composite	Whole	<0.722	<0.723	<0.7225
500-6730	Core 52 composite	Whole	0.291	0.184	0.2375 ^{QC:c}
Solids: water digest			$\mu\text{Ci/g}$	$\mu\text{Ci/g}$	$\mu\text{Ci/g}$
534-7730	Core 51 composite	Whole	<0.0324	<0.0327	<0.03255
534-9730		Whole	0.0371	<0.0338	<0.03545
528-7730	Core 52 composite	Whole	<0.0649	<0.0653	<0.0651
528-9730		Whole	0.0383	0.0375	0.0379
Liquids			$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$
393-5782	Core 50 composite	Drainable liquid	<0.013	<0.0129	<0.01295
442-5782	Core 51 composite	Drainable liquid	<0.0304	<0.0304	<0.0304
494-5782	Core 52 composite	Drainable liquid	0.00373	<0.00335	<0.00354

Table B2-79. Tank 241-T-107 Analytical Results: Ruthenium-103 (GEA).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion			$\mu\text{Ci/g}$	$\mu\text{Ci/g}$	$\mu\text{Ci/g}$
387-6730	50: 1R	Subsegment	<0.0112	<0.0113	<0.01125
309-6730	50: 2	Whole	<0.0216	<0.0206	<0.0211
322-6730	50: 3	Whole	<0.0104	<0.0112	<0.0108
406-6730	51: 2	Whole	<0.0613	<0.0628	<0.06205
255-6730	51: 3	Lower half	<0.0264	<0.0271	<0.02675
254-6730		Upper half	<0.0249	<0.0236	<0.02425
257-6730	51: 4	Lower half	<0.0233	<0.0226	<0.02295
256-6730		Upper half	<0.0271	<0.0274	<0.02725
455-6730	52: 1	Whole	<0.0146	<0.0142	<0.0144
481-6730	52: 2	Whole	<0.0383	<0.034	<0.03615
483-6730	52: 3	Lower half	<0.0363	<0.0356	<0.03595
482-6730		Upper half	<0.0337	<0.0341	<0.0339
436-6782	Core 51 composite	Whole	<0.04	<0.04	<0.04
500-6730	Core 52 composite	Whole	<0.0129	<0.0137	<0.0133
Solids: water digest			$\mu\text{Ci/g}$	$\mu\text{Ci/g}$	$\mu\text{Ci/g}$
534-7730	Core 51 composite	Whole	<0.00619	<0.00593	<0.00606
534-9730		Whole	<0.0256	<0.0278	<0.0267
528-7730	Core 52 composite	Whole	<0.00711	<0.00721	<0.00716
528-9730		Whole	<0.0142	<0.0155	<0.01485
Liquids			$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$
393-5782	Core 50 composite	Drainable liquid	<0.0015	<0.00152	<0.00151
442-5782	Core 51 composite	Drainable liquid	<0.00717	<0.00734	<0.007255
494-5782	Core 52 composite	Drainable liquid	<0.00127	<0.0013	<0.001285

Table B2-80. Tank 241-T-107 Analytical Results: Ruthenium/Rhodium-106 (GEA).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion			$\mu\text{Ci/g}$	$\mu\text{Ci/g}$	$\mu\text{Ci/g}$
387-6730	50: 1R	Subsegment	<0.163	<0.153	<0.158
309-6730	50: 2	Whole	<0.326	<0.311	<0.3185
322-6730	50: 3	Whole	<0.16	<0.166	<0.163
406-6730	51: 2	Whole	<0.618	<0.653	<0.6355
255-6730	51: 3	Lower half	<0.297	<0.288	<0.2925
254-6730		Upper half	<0.288	<0.268	<0.278
257-6730	51: 4	Lower half	<0.257	<0.253	<0.255
256-6730		Upper half	<0.285	<0.303	<0.294
455-6730	52: 1	Whole	---	<0.21	<0.105 ^{QCc}
481-6730	52: 2	Whole	<0.503	<0.503	<0.503
483-6730	52: 3	Lower half	<0.474	<0.479	<0.4765
482-6730		Upper half	<0.439	<0.428	<0.4335
436-6782	Core 51 composite	Whole	<0.495	<0.56	<0.5275
500-6730	Core 52 composite	Whole	<0.175	<0.193	<0.184
Solids: water digest			$\mu\text{Ci/g}$	$\mu\text{Ci/g}$	$\mu\text{Ci/g}$
534-7730	Core 51 composite	Whole	<0.0791	<0.0757	<0.0774
534-9730		Whole	<0.161	<0.181	<0.171
528-7730	Core 52 composite	Whole	<0.0752	<0.0749	<0.07505
528-9730		Whole	<0.0957	<0.0986	<0.09715
Liquids			$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$
393-5782	Core 50 composite	Drainable liquid	<0.0208	<0.0207	<0.02075
442-5782	Core 51 composite	Drainable liquid	<0.0932	<0.0951	<0.09415
494-5782	Core 52 composite	Drainable liquid	<0.0163	<0.0168	<0.01655

Table B2-81. Tank 241-T-107 Analytical Results: Thorium-228 (GEA).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion			$\mu\text{Ci/g}$	$\mu\text{Ci/g}$	$\mu\text{Ci/g}$
387-6730	50: 1R	Subsegment	<0.0167	<0.0167	<0.0167
309-6730	50: 2	Whole	<0.0349	<0.0333	<0.0341
322-6730	50: 3	Whole	<0.0179	<0.0193	<0.0186
406-6730	51: 2	Whole	<0.088	<0.0892	<0.0886
255-6730	51: 3	Lower half	<0.043	<0.045	<0.044
254-6730		Upper half	<0.0421	<0.0385	<0.0403
257-6730	51: 4	Lower half	<0.0346	<0.0332	<0.0339
256-6730		Upper half	<0.0423	<0.0442	<0.04325
455-6730	52: 1	Whole	<0.0235	<0.0227	<0.0231
481-6730	52: 2	Whole	<0.0478	<0.0462	<0.047
483-6730	52: 3	Lower half	<0.0458	<0.0455	<0.04565
482-6730		Upper half	<0.0457	<0.0439	<0.0448
436-6782	Core 51 composite	Whole	0.0513	<0.0505	<0.0509
500-6730	Core 52 composite	Whole	<0.0205	<0.0214	<0.02095
Solids: water digest			$\mu\text{Ci/g}$	$\mu\text{Ci/g}$	$\mu\text{Ci/g}$
534-7730	Core 51 composite	Whole	<0.00873	<0.00844	<0.008585
534-9730		Whole	<0.0518	<0.0579	<0.05485
528-7730	Core 52 composite	Whole	<0.0103	<0.0107	<0.0105
528-9730		Whole	<0.0295	<0.031	<0.03025
Liquids			$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$
393-5782	Core 50 composite	Drainable liquid	<0.00208	<0.0021	<0.00209
442-5782	Core 51 composite	Drainable liquid	<0.0102	<0.0103	<0.01025
494-5782	Core 52 composite	Drainable liquid	<0.00178	<0.00184	<0.00181

Table B2-82. Tank 241-T-107 Analytical Results: Iodine-129 (Low Energy GEA).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Liquids			$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$
393-5785	Core 50 composite	Drainable liquid	< 7.480E-05	< 8.260E-05	< 7.870E-05 ^{QC:a}
442-5785	Core 51 composite	Drainable liquid	< 3.850E-05	< 3.720E-05	< 3.785E-05 ^{QC:c}
494-5785	Core 52 composite	Drainable liquid	< 3.790E-05	< 3.840E-05	< 3.815E-05 ^{QC:c}

Table B2-83. Tank 241-T-107 Analytical Results: Carbon-14 (Liquid Scintillation).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: water digest			$\mu\text{Ci/g}$	$\mu\text{Ci/g}$	$\mu\text{Ci/g}$
430-7788	Core 51 composite	Whole	2.550E-04	< 2.220E-04	< 2.385E-04
506-7788	Core 52 composite	Whole	1.150E-04	1.730E-04	1.440E-04 ^{QC:c}
Liquids			$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$
393-5788	Core 50 composite	Drainable liquid	1.990E-05	1.600E-05	1.795E-05 ^{QC:c}
442-5788	Core 51 composite	Drainable liquid	3.800E-04	6.560E-04	5.18E-04
494-5788	Core 52 composite	Drainable liquid	6.220E-05	8.550E-05	7.385E-05 ^{QC:c}

Table B2-84. Tank 241-T-107 Analytical Results: Tritium (Liquid Scintillation).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: water digest			$\mu\text{Ci/g}$	$\mu\text{Ci/g}$	$\mu\text{Ci/g}$
430-7787	Core 51 composite	Whole	0.00156	0.00117	0.001365 ^{QC,d,c}
430-7787		Whole	< 6.150E-04	8.850E-04	< 7.500E-04 ^{QC,c}
506-7787	Core 52 composite	Whole	0.00104	0.00118	0.00111
Liquids			$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$
393-5787	Core 50 composite	Drainable liquid	5.050E-04	2.650E-04	3.850E-04 ^{QC,e}
393-5787		Drainable liquid	2.480E-04	3.430E-04	2.955E-04 ^{QC,a,e}
442-5787	Core 51 composite	Drainable liquid	0.00214	0.00231	0.002225 ^{QC,c}
442-5787		Drainable liquid	0.00993	0.00396	0.006945 ^{QC,c}
494-5787	Core 52 composite	Drainable liquid	4.890E-04	4.520E-04	4.705E-04

Table B2-85. Tank 241-T-107 Analytical Results: Technetium-99 (Liquid Scintillation).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: fusion			$\mu\text{Ci/g}$	$\mu\text{Ci/g}$	$\mu\text{Ci/g}$
436-6784	Core 51 composite	Whole	0.046	0.0505	0.04825 ^{QC,c}
500-6784	Core 52 composite	Whole	0.0512	0.0543	0.05275 ^{QC,c}
Liquids			$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$	$\mu\text{Ci/mL}$
393-5784	Core 50 composite	Drainable liquid	0.00793	0.00816	0.008045 ^{QC,c}
442-5784	Core 51 composite	Drainable liquid	0.0858	---	0.0858 ^{QC,a,c}
494-5784	Core 52 composite	Drainable liquid	0.0171	0.0162	0.01665 ^{QC,a,c}

Table B2-86. Tank 241-T-107 Analytical Results: Bulk Density.

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids			g/mL	g/mL	g/mL
535-1755	50: 2	Whole	1.71	---	1.71
93-07217		Whole	1.44	---	1.44
93-07217		1:3 dilution	1.10	---	1.1
93-07217		1:1 dilution	1.22	---	1.22
283-1755	51: 3	Lower half	1.70	---	1.7
282-1755		Upper half	1.49	---	1.49
285-1755	51: 4	Lower half	1.53	---	1.53
284-1755		Upper half	1.48	---	1.48
517-1755	52: 2	Whole	1.55	---	1.55
519-1755	52: 3	Lower half	1.52	---	1.52
518-1755		Upper half	1.50	---	1.50
420-1755	Core 51 composite	Whole	1.46	---	1.46
546-1755	Core 52 composite	Whole	1.19	---	1.19

Table B2-87. Tank 241-T-107 Analytical Results: Density.¹

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids			g/mL	g/mL	g/mL
n/a	50: 1R	Subsegment	1.71	---	1.71
n/a		Subsegment	1.76	---	1.76
n/a	50: 2	Whole	1.1	---	1.1
n/a	50: 3	Whole	0.95	---	0.95
n/a	51: 2	Whole	1.57	---	1.57
n/a	51: 3	Whole	1.15	---	1.15
n/a	51: 4	Whole	1.1	---	1.1
n/a	52: 1	Whole	1.42	---	1.42
n/a	52: 2	Whole	1.06	---	1.06
n/a	52: 3	Whole	1.13	---	1.13
n/a	52: 4	Whole	0.61	---	0.61
Liquids			g/mL	g/mL	g/mL
n/a	50: 3	Drainable liquid	0.96	---	0.96
n/a	50: 4	Drainable liquid	0.97	---	0.97
n/a	51: 2	Drainable liquid	1.26	---	1.26
n/a	52: 4	Drainable liquid	1.12	---	1.12

Note: n/a = not applicable

¹Estimations from hot cell, not analytical data.

Table B2-88. Tank 241-T-107 Analytical Results: Specific Gravity.

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Liquids			unitless	unitless	unitless
393-5706	Core 50 composite	Drainable liquid	1.02	1.02	1.02
442-5706	Core 51 composite	Drainable liquid	1.21	---	1.21
442-5806		Drainable liquid	1.19	---	1.19
494-5706	Core 52 composite	Drainable liquid	1.1	1.11	1.105

Table B2-89. Tank 241-T-107 Analytical Results: pH Measurement.

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: water digest			Unitless	Unitless	Unitless
312-5715	50: 1R	Subsegment	10.3	---	10.3
313-5715	50: 2	Whole	11.2	---	11.2
122-5715	50: 3	Whole	11.4	---	11.4
411-5715	51: 2	Whole	10.6	10.5	10.55
211-5715	51: 3	Lower half	11.4	11.3	11.35
210-5715		Upper half	11.4	11.4	11.4
213-5715	51: 4	Lower half	11.6	11.6	11.6
212-5715		Upper half	11.2	11.2	11.2
455-5715	52: 1	Whole	10.5	10.5	10.5
468-5715	52: 2	Whole	11.4	11.4	11.4
470-5715	52: 3	Lower half	10.9	10.8	10.85
469-5715		Upper half	11.8	11.8	11.8
427-5715	Core 51 composite	Whole	11.6	11.6	11.6
495-5715	Core 52 composite	Whole	11.4	11.4	11.4
Liquids			Unitless	Unitless	Unitless
393-5713	Core 50 composite	Drainable liquid	9.63	9.62	9.625
442-5713	Core 51 composite	Drainable liquid	10.7	10.7	10.7
494-5713	Core 52 composite	Drainable liquid	10.3	10.3	10.3

Table B2-90. Tank 241-T-107 Analytical Results: Weight Percent Solids (Gravimetry).

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids			%	%	%
314-5710	50: 1R	Subsegment	82.5	81.5	82
315-5710	50: 2	Whole	58.3	58.6	58.45
415-5710	51: 2	Whole	40.3	39.2	39.75
269-5710	51: 3	Lower half	46.7	47.5	47.1
268-5710		Upper half	44	45.7	44.85
271-5710	51: 4	Lower half	50.1	50.8	50.45
93-7224-K1		Lower half	49.5	49.8	49.65
270-5710		Upper half	44.5	45.6	45.05
471-5710	52: 1	Whole	83.4	83.3	83.35
457-5710	52: 2	Whole	51.1	51.9	51.5
459-5710	52: 3	Lower half	46.6	46.5	46.55
458-5710		Upper half	48.5	48.7	48.6
421-5710	Core 51 composite	Whole	48.6	47.6	48.1
510-5710	Core 52 composite	Whole	52.2	52.3	52.25
93-7224-K1		Whole	56.3	58.5	57.4

Table B2-91. Tank 241-T-107 Analytical Results: Weight Percent Residual Solids.

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: water digest			%	%	%
534-9210	Core 51 composite	Whole	27.7	24.2	25.95
528-9710	Core 52 composite	Whole	30.8	32.2	31.5

Table B2-92. Tank 241-T-107 Analytical Results: Residual Solids.

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids: water digest			g	g	g
534-9210	Core 51 composite	Whole	0.1429	0.1254	0.13415
528-9710	Core 52 composite	Whole	0.1529	0.1577	0.1553

Table B2-93. Tank 241-T-107 Analytical Results: Total Dissolved Solids.

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids			%	%	%
534-7705	Core 51 composite	Whole	0.378	0.396	0.387
528-7705	Core 52 composite	Whole	0.412	0.392	0.402
Liquids			%	%	%
393-5705	Core 50 composite	Drainable liquid	4.1	4.67	4.385
442-5705	Core 51 composite	Drainable liquid	24	25.4	24.7
494-5705	Core 52 composite	Drainable liquid	13.2	13.7	13.45

Table B2-94. Tank 241-T-107 Analytical Results: Shear Strength.

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids			dynes/cm²	dynes/cm²	dynes/cm²
93-072167	50: 2	Whole	7,200	N/A	7,200

Table B2-95. Tank 241-T-107 Analytical Results: Rheological Parameters.

Sample Number	Sample Location	Sample Portion	Yield Point	Consistency Factor	Flow Behavior Index
Solids			Pa	Pa/sec	Unitless
93-07217 (25 °C)	50: 2	1:1 dilution	0.70	0.024	0.78
93-07217 (25 °C)		1:1 dilution	0.60	0.027	0.76
93-07217 (90 °C)		1:1 dilution	0.89	0.025	0.74
93-07217 (90 °C)		1:1 dilution	0.66	0.019	0.73

Table B2-96. Tank 241-T-107 Analytical Results: Percent Water (TGA). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Solids			%	%	%
306-5712	50: 1R	Subsegment	27	25.4	26.2
384-5712		Subsegment	5.87	5.65	5.76
121-5712	50: 2	Whole	29.6	29.9	29.75
307-5712		Whole	45.8	40.3	43.05
122-5712	50: 3	Whole	44.4	42.1	43.25
123-5712	50: 4	Whole	57.2	59	58.1
402-5712	51: 2	Whole	60.1	58.5	59.3
157-5712	51: 3	Lower half	54.3	54.2	54.25
156-5712		Upper half	59.4	59.9	59.65
159-5712	51: 4	Lower half	52.8	53.4	53.1
158-5712		Upper half	54.8	54.6	54.7
455-5712	52: 1	Whole	15.2	15.3	15.25
485-5712	52: 2	Whole	55.6	55.4	55.5
487-5712	52: 3	Lower half	54.8	49.7	52.25
486-5712		Upper half	54.6	54.6	54.6
456-5712	52: 4	Whole	59.3	57.7	58.5

Table B2-96. Tank 241-T-107 Analytical Results: Percent Water (TGA). (2 sheets)

Sample Number	Sample Location	Sample Portion	Result	Duplicate	Mean
Liquids			%	%	%
393-5712	Core 50 composite	Drainable liquid	96.5	93.7	95.1
442-5712	Core 51 composite	Drainable liquid	73.4	74	73.7
494-5712	Core 52 composite	Drainable liquid	83.2	82.6	82.9

B2.2 OCTOBER 1992 VAPOR SAMPLES

B2.2.1 Description of Sampling Event

Before the November 1992 core sampling of tank 241-T-107, a vapor measurement was taken on October 22, 1992. Testing for flammable gas levels, which was driven by tank farm operations, was a necessary step before beginning core sampling procedures. The vapor samples were taken to measure tank headspace flammability and concentrations of ammonia, organics, NO, NO₂, N₂O, O₂, H₄N₂, CN⁻, and HCN. The vapor measurements were taken at three levels in the tank headspace; short (sample line length of 17 ft 6 in.), medium (line length of 24 ft 3 in. - approximately half the distance), and long (31 ft 8 in. - 1 ft above waste level). All results were obtained in the field (that is, no gas samples were sent to the laboratory for analysis). The results were reported in *Waste Tank T-107 Vapor Sampling Results* (Pingel 1992).

B2.2.2 Sample Handling

Pingel (1992) gives few details of sample handling except for temperature and barometric pressure. Refer to the field log book for details (WHC-N-499, Vol. 1).

B2.2.3 Sample Analysis

Descriptions of the sampling and analysis for the vapor samples are reported in Pingel (1992). A portable combustible gas meter was used for the flammability measurement. An organic vapor monitor was used to sample total organics in the gas sample stream. Calorimetric sorbent tubes were used for detecting the remaining gases.

B2.2.4 Analytical Results

Table B2-97 summarizes the 1992 results of the vapor sampling event. All gases tested with the calorimetric sorbent tubes were below the detection limits except ammonia. Except for the total organics vapor results, there was no variance between the results from differing levels within the tank headspace. Table B2-97 shows the highest results of the organic vapor measurements. This summary is taken from Pingel (1992).

Table B2-97. Summary Results of Vapor Samples Collected from the Headspace of Tank 241-T-107 on October 22, 1992.

Category	Analyte	Vapor Concentration
Inorganic	O ₂	20.9 %
	NH ₃	203 ppmv
	NO ₂	≤ 0.5 ppmv
	NO	≤ 0.5 ppmv
	HCN	<2 ppmv
	CN	<2 ppmv
	H ₄ N ₂	<0.2 ppmv
Organic	Total organic vapors	42 ppmv
Flammability	Headspace gas flammability	0% of LEL

B2.3 JANUARY 1995 VAPOR SAMPLES

B2.3.1 Description of Sampling Event

To support the safety screening DQO (Dukelow et al. 1995) and the vapor DQO (Osborne et al. 1994), additional vapor phase measurements were made on January 18, 1995. The vapor phase screening was taken for flammability and toxicity issues. The results were reported in *Tank 241-T-107 Headspace Gas and Vapor Characterization Results for Samples Collected in January 1995* (Huckaby and Bratzel 1995). The vapor phase measurements were taken 5.5 m (18 ft) below riser 6 in the headspace of the tank, and the gas samples were sent to the laboratory for analysis.

B2.3.2 Sample Handling

Sampling devices, including three sorbent trains (for inorganic analyses) and four SUMMA™ canisters (for organic analyses), were supplied to the Westinghouse Hanford Company sampling staff, and the samples were collected. Sampling media were prepared and analyzed by Pacific Northwest National Laboratory and Oak Ridge National Laboratory. Detailed descriptions of the sampling and analysis of these vapor samples are reported in *Vapor Space Characterization of Waste Tank 241-T-107: Results from Samples Collected on 1/18/95* (Pool et al. 1995).

B2.3.3 Sample Analysis

Designated holding times of less than 60 days before analysis were met. Sample analysis is covered in depth in Huckaby and Bratzel (1995).

B2.3.4 Analytical Results

It was determined that no headspace constituents exceeded the flammability or industrial hygiene notification limits specified in the *Tank 241-T-107 Tank Characterization Plan* (Carpenter 1995; Huckaby and Bratzel 1995). Table B2-98 summarizes the results. For the complete list of tentatively and positively identified organic compounds, see Huckaby and Bratzel (1995).

Table B2-98. Summary Results of Vapor Samples Collected from the Headspace of Tank 241-T-107 on January 18, 1995.¹ (2 sheets)

Category	Analyte	Vapor Concentration	Units
Inorganic	Hydrogen	< 94	ppmv
	NH ₃	125	ppmv
	CO	< 12	ppmv
	CO ₂	75	ppmv
	NO	< 0.05	ppmv
	NO ₂	< 0.03	ppmv
	N ₂ O	41.5	ppmv
	H ₂ O	12.1	mg/L
	H ₂ O	82 %	% relative humidity

Table B2-98. Summary Results of Vapor Samples Collected from the Headspace of Tank 241-T-107 on January 18, 1995.¹ (2 sheets)

Category	Analyte	Vapor Concentration	Units
Organic ²	Methane	< 61	ppmv
	Tributyl phosphate	unknown ³	ppmv
	n-Dodecane	0.0021	ppmv
	n-Tridecane	0.0056	ppmv
	Total estimated organic vapor	1.4 ⁴	mg/m ³
Flammability	Overall headspace gas flammability	No overall result was given in Huckaby and Bratzel (1995). Document states all results were below vapor DQO limit (20 percent of LFL ⁵).	

Notes:

¹Huckaby and Bratzel (1995)²Summary of key constituents only; complete list can be found in Huckaby and Bratzel (1995).³The absence may be due to sampling methods.⁴This value is the summation of quantitated and estimated organic vapor concentrations in samples analyzed at Oak Ridge National Laboratory.⁵This limit has since been increased to 25 percent of the LFL (Osborne and Buckley 1995).**B2.4 HISTORICAL SAMPLE RESULTS**

Historical sampling data for tank 241-T-107 is available from three samplings of the tank liquid. The dates of sampling are September 22, 1965; March 5, 1985; and August 1, 1989. No information was available about sample handling or analysis for the samples. The reasons for the samplings were given as evaporator feed and mixing studies. The samples were reported as being a clear and having an amber color with few solids.

Table B2-99 shows the results of the 1965 sample (Godfrey 1965). Because the tank has been pumped of liquids, the sample probably does not represent the current tank contents.

Table B2-99. September 1965 Supernatant Sample.¹

Analysis of Tank 241-T-107 Sample		
September 24, 1965		
COMPONENT	LAB VALUE	LAB UNIT
Physical Data		
Sample description	Clear, amber, no solids. 300 mrad/hr.	
Specific gravity	1.204	Unitless
Chemical Analysis		
AlO ₂	0.20	g/L
NaOH	0.164	N
NO ₃	203.5	g/L
CO ₃	13.5	g/L
Cl	10.5	g/L
Na	98	g/L
Radiological Analysis		
¹³⁷ Cs	7,700	μCi/L

Note:

N = normal

¹Pre-1989 analytical data have not been validated and should be used with caution.

Table B2-100 shows the results of the 1975 sample. Because the sample was liquid and the tank has been pumped of liquids, the sample probably does not represent current tank contents. In addition to the sample analysis, cooling curve measurements were performed on the sample (Wheeler 1975).

Table B2-101 shows the results of the 1985 sample. Because the sample was liquid and the tank has been pumped of liquids, the sample probably does not represent current tank contents. The sample was analyzed for specific constituents, and the results were reported in *Tank 107-T Waste Mixing Study* (Bratzel 1985).

Table B2-100. September 1975 Supernatant Sample.¹

Supernatant Sample Number: T-8248		
Sample Received: September 25, 1975		
COMPONENT	LAB VALUE	LAB UNIT
Physical Analysis		
Sample Description	Dark yellow, < 1 percent solids, 1 rad/hr	
pH	12.3	---
Specific gravity	1.129	---
DTA	no exotherm	---
Water	85.08	%
Chemical Analysis		
OH	0.0807	M
Al	0.00532	M
Na	2.44	M
NO ₂	0.651	M
NO ₃	0.800	M
PO ₄	0.0251	M
Cl	0.0128	M
F	0.0135	M
CO ₃	0.394	M
Radiological Analysis		
Pu	< 3.78E-06	g/gal
¹⁰⁶ RuRh	3.21E+05	μCi/gal
¹³⁴ Cs	869	μCi/gal
¹³⁷ Cs	1.48E+05	μCi/gal
^{89,90} Sr	8,840	μCi/gal
Cooling Curve Data		
35 °C for 35 minutes		No solids
30 °C for 35 minutes		No solids
25 °C for 35 minutes		No solids
20 °C for 35 minutes		No solids
15 °C for 35 minutes		No solids
10 °C for 35 minutes		No solids
5 °C for 35 minutes		No solids

Note:

DTA = differential thermal analysis

¹Pre-1989 analytical data have not been validated and should be used with caution.

Table B2-101. March 1985 Supernatant Sample.¹

Tank 241-T-107 Mixing Study		
Supernatant Sample Number: R-3872		
COMPONENT	LAB VALUE	LAB UNIT
Chemical Analysis		
La	< 1.53E-04	M
Ta	< 2.96E-04	M
Bi	< 5.32E-04	M
Mo	< 7.97E-04	M
Ba	< 3.68E-05	M
Zr	< 5.09E-04	M
Zn	< 7.72E-05	M
Sr	< 1.15E-05	M
NO ₂	0.546	M
SO ₄	0.178	M
CO ₃	0.358	M
F	< 0.00379	M
TOC	0.924	g C/L
U	0.057	g/L
Radiological Analysis		
^{239/40} Pu	9.84	μCi/L
²⁴¹ Am	< 0.317	μCi/L
⁸⁹ Sr	294	μCi/L
¹³⁷ Cs	25,000	μCi/L

Note:

g C/L = grams of carbon per liter

¹Pre-1989 analytical data have not been validated and should be used with caution.

Table B2-102 shows the results of the 1989 sample. Because the sample was liquid and the tank has since been pumped of liquids, the sample probably does not represent current tank contents. The sample was analyzed for specific constituents, and the results showed high concentrations of sodium and nitrate. The radionuclides tested for were cesium, strontium, plutonium, and americium (Bratzel 1989).

Table B2-102. August 1989 Supernatant Sample.

Analysis of Tank Farm Sample: August 1, 1989		
Interim Results		
COMPONENT	LAB VALUE	LAB UNIT
Physical Data		
pH	11.0	---
Specific gravity	1.20	---
Chemical Analysis		
Al	0.00102	M
F	8.24E-0	M
Na	2.35	M
NO ₂	0.583	M
NO ₃	2.19	M
OH	0.0249	M
CO ₃	0.374	M
SO ₄	0.170	M
PO ₄	0.0616	M
F	0.0491	M
Cl	0.0354	M
TOC	0.864	M
P	0.0450	M
K	0.00638	M
Cr	0.00408	M
B	0.00127	M
Radiological Analysis		
^{239/240} Pu	12.0	μCi/L
²⁴¹ Am	0.0199	μCi/L
Total alpha	62.0	μCi/L
Total beta	28,800	μCi/L
⁸⁹ Sr	331	μCi/L
¹³⁷ Cs	22,300	μCi/L

B3.0 ASSESSMENT OF CHARACTERIZATION RESULTS

This section discusses the overall quality and consistency of the current sampling results for tank 241-T-107 and shows the results of the calculation of tank mean concentrations.

This section also evaluates sampling and analysis factors that may impact data interpretation. These factors are used to assess data overall quality and consistency and to identify limitations in data use.

B3.1 FIELD OBSERVATIONS

Three core samples (cores 50, 51, and 52) were taken from tank 241-T-107. Each core was expected to consist of four segments. The top segment was expected to be one-quarter full. Several segments experienced poor recoveries and others had good overall recoveries but few solids. For example, segments 3 and 4 of core 50 had overall recoveries of 96 and 67 percent, respectively. Segment 3 was 95 percent liquid, and segment 4 was 99 percent liquid. In fact, there were insufficient solids from core 50 to make a solids composite. Because the overall tank means are based on the core composite data, no waste from core 50 figured in the mean calculations for the solids.

Except for segment 1 (which was empty), core 51 had the best recoveries. The poor segment recoveries also impacted the segment analyses, because insufficient sample was available for a full suite of analyses for some segments. Only three segments could be split into upper and lower halves for analysis because of the poor solids recoveries. The impact of the poor recoveries on data quality is unknown. There are concerns that the full waste depth profile was not recovered, and the data may not fully represent tank waste.

It is not known whether the high liquid compositions of some segments were caused by intrusions into the samples from the hydrostatic head fluid. Water was used as a hydrostatic head fluid. However, because no tracer was added, contamination of the segments by the head fluid could not be tracked. Based on visual observation, the 222-S Laboratory indicated no contamination was apparent.

Problems were also encountered during sampling and extrusion. Core 50, segment 1, remained in the riser for more than 48 hours. Because each sampling event has limits for time elapsed between sampling and delivery of the sample to the laboratory, this requirement was violated, and core 50, segment 1, was resampled (designated 1R). During extrusion, the sampler for core 50, segment 2, was under pressure. When it was opened, a small amount of sample was ejected; however, a sufficient amount of sample remained (194.6 g) to perform all analyses. The segments for tank 241-T-107 were broken down according to Bell (1993).

B3.2 QUALITY CONTROL ASSESSMENT

A QC assessment was performed on the data from the last two sampling events. Section B3.2.1 discuss the QC results from the 1992 and 1993 core sampling event, and Section B3.2.2 discusses the QC results from the 1995 vapor sampling event.

B3.2.1 Quality Control Assessment for the 1992 and 1993 Core Sampling Event

The usual QC assessment for solid and liquid samples includes an evaluation of the appropriate standard recoveries, spike recoveries, duplicate analyses, and blanks that are performed in conjunction with the chemical analyses. All the pertinent quality control tests were conducted on the 1992 and 1993 core samples, enabling a full assessment of data accuracy and precision. Bell (1993) established specific criteria for all analytes. Sample and duplicate pairs, with one or more QC results outside the specified criteria, were identified by footnotes in the data summary tables. Major analytes, which met all QC criteria, included cyanide, nitrite, sulfate, TOC, and weight percent water. The following assessment focuses primarily on major analytes and analytes of concern.

The standard and spike recovery results provide an estimate of analysis accuracy. If a standard or spike recovery is above or below the given criterion, the analytical results may be biased high or low, respectively. Many analytes had minor deviations from the standard recovery criteria, but these were characteristic of variations in the rate of sample introduction caused by the high salt content of the samples. One of the spike recoveries was outside the target level for total alpha activity. This may have been caused by high dissolved solids content on the sample mount and subsequent self-shielding. Spike recoveries outside the limits for sodium and other major ICP analytes such as aluminum, bismuth, iron, and phosphorus were probably caused by the high dilutions required. These high dilutions in turn can cause poor or meaningless spike recoveries and RPDs for ICP elements that had very high concentrations or were close to the detection limit. Low recoveries for many analytes were caused by matrix effects. The high spike recoveries for boron and silicon were caused by hydrofluoric acid in the standard matrix reacting with the glassware.

Precision is estimated by the RPD, which is defined as the absolute value of the difference between the primary and duplicate samples, divided by their mean, times 100. Three total alpha activity samples had high RPDs probably caused by low sample activity and self-adsorption. The RPDs were exceeded for many analytes with concentrations near the detection limit, because this adversely impacts the result reproducibility results. Some high RPDs may be attributable to sample homogeneity problems.

Contamination generally was not a problem. Silicon was noted above the detection limit in some blanks, but this was caused by contamination from the glassware. Sodium was detected at very low concentrations in the blanks, but this was attributed to sample carryover and was inconsequential when compared to the analyte concentration.

A complete validation was performed on the data. This validation included a detailed examination of the data package to recreate the analytical process and verify that proper and acceptable analytical techniques had been applied. Additionally, the data package was checked for correct submission of required deliverables and summary forms and for proper calculation of parameters (Svancara and Pool 1993).

Only 75 analytical results, all ICP data, were rejected. In 42 cases, matrix spike failure was the reason for rejection. As previously discussed, spike recoveries can be poor for analytes near the detection limit because of the high dilutions required. Most analytes with matrix spike recovery problems fell into this category. Boron and silicon experienced matrix spike recovery problems for the same reason.

All 13 nickel fusion results were rejected because of contamination from the nickel crucibles used during the fusion. Other reasons for rejecting data included laboratory control sample failure, calibration verification problems, and duplicate analysis failure. The rejected data are noted with an "r" in the analytical tables in the "Mean" column. Twenty-nine rejected results were below the detection limits.

In summary, the majority of QC results for the core samples were within boundaries specified by Bell (1993). The discrepancies mentioned here and footnoted in the data summary tables should not impact the validity or the use of data for the purpose of evaluating the requirement of the safety screening DQO.

B3.2.2 Quality Control Assessment for the 1995 Vapor Sampling Event

The only QC information provided in Huckaby and Bratzel (1995) were relative standard deviations (RSDs) for each analyte. Although the Tank Characterization Plan (Carpenter 1995) specified certain QC criteria, it appears from Huckaby and Bratzel (1995) that the QC stipulations in Burnum (1995) were followed. Burnum (1995) specified that the RSDs should be less than 25 percent. The RSD is a measure of variability defined as the standard deviation divided by the mean, times 100.

Positive identification of organic analytes involves matching the gas chromatography (GC) retention times and mass spectrometry (MS) data from a sample with that obtained from analysis of known compounds. The concentration of an analyte in the sample is said to be quantitatively measured if the response of the GC/MS has been established at several known concentrations of that analyte (the GC/MS has been calibrated for that analyte), and the MS response to the analyte in the sample is between the lowest and highest responses to the known concentrations (the analyte is within the calibration range). In this QC summary, only those gases defined as inorganic or those organic gases defined as quantitatively measured or positively identified will be assessed (Huckaby and Bratzel 1995).

Five inorganic gases were detected, and all met the QC criteria. Six organic gases were defined as quantitatively measured, and four met the criteria. The remaining two had RSDs of 33 percent and 37 percent. Nineteen organic analytes, which did not exceed their holding times, were positively identified, but the results cannot be considered quantitative and may not be accurate to within the ≤ 25 percent criteria established by Burnum (1995). Nine of these 19 gases exceeded the QC criteria (Huckaby and Bratzel 1995).

B3.3 DATA CONSISTENCY CHECKS

Comparing different analytical methods can help to assess data consistency and quality. Several comparisons were possible with the data set provided by the core samples. These included a comparison of phosphorus and sulfur as analyzed by ICP with phosphate and sulfate as analyzed by IC, a comparison of total alpha activity and total beta activity with the sum of the individual alpha and beta emitters, respectively, and a comparison of the moisture content results from the TGA and gravimetric determinations. In addition, mass and charge balances were calculated to help assess the overall data consistency.

B3.3.1 Comparison of Results from Different Analytical Methods

The data consistency checks below compare results from two analytical methods. Close agreement between the methods strengthens the credibility of both results, but poor agreement brings the reliability of the data into question. All analytical mean results were taken from Table B3-6. In cases where more than one digestion method was conducted for a given analyte, the method that yielded the highest analytical result was used for comparison.

The IC phosphate mean result was 114,000 $\mu\text{g/g}$. The analytical phosphorus mean result, as determined by the ICP fusion digest results, was 32,900 $\mu\text{g/g}$. This is equivalent to 101,000 $\mu\text{g/g}$ of phosphate and yields a ratio of 1.14. The IC results indicate more phosphorus in the tank than the ICP results do. Furthermore, the mean water digest ICP result for phosphorus is 23,700 $\mu\text{g/g}$, equivalent to 72,600 $\mu\text{g/g}$ of phosphate. Comparison with the fusion result indicates that 72 percent of the phosphorous is water soluble; therefore, inconsistencies exist between the ICP and IC results for phosphorus/phosphate. The IC results may be biased high, or the ICP results may be low.

The analytical sulfur mean result, as determined by the ICP fusion digest results, was 3,140 $\mu\text{g/g}$, equivalent to 9,420 $\mu\text{g/g}$ of sulfate. The IC sulfate mean result was 9,970 $\mu\text{g/g}$, yielding a ratio between the two methods of 0.94. This comparison indicates that nearly all sulfate in the tank is water soluble. A second check for solubility is to compare the ICP fusion and water digest means for sulfur. The water digest sulfur mean was 3,540 $\mu\text{g/g}$, yielding a ratio of 1.08 with the fusion mean. This comparison supported the belief that all sulfur is present in a water soluble form.

The total alpha and total beta activities can be compared to the sums of the activities of the individual alpha and beta emitters. The fusion digested results from the composite samples were used in all cases.

The sum of activities of individual alpha emitters was 0.172 $\mu\text{Ci/g}$, determined by adding the ^{241}Am and $^{239/240}\text{Pu}$ mean activities (the two major alpha emitters) and the ^{238}U activity calculated from laser fluorimetry data. The gross alpha activity was 0.434 $\mu\text{Ci/g}$, yielding a ratio between the two methods of 0.40. There is a large discrepancy between the two methods. Total alpha results were difficult to obtain because of interference from the high solids on the sample mounts resulting from the fusion preparation. Therefore, small sample sizes were used to minimize the amount of solids on the mount. Normally, plutonium and americium account for >95 percent of the total alpha results. However, as the comparison reveals, this is not true for composite fusion digestion samples. The results appear to show a higher total alpha concentration than the sum of the representative isotopes. The higher total alpha concentration may be caused by the following.

1. High counting error
2. Interference from ^{137}Cs and $^{90}\text{Sr}/^{90}\text{Y}$ present in the samples

(The total beta was over 700 times greater than total alpha for these samples.) A small amount of the β -emissions may have been confounding the detector. (The activity of the samples is so low that the offset used to discriminate between alpha and beta plateaus was not sufficient to provide accurate readings.) The issue of interference between alpha and beta emitters has been resolved since.

3. Another alpha emitting isotope may be present which is not identified or quantified.

Isotopic determination of the samples was obtained by thermal ionization mass spectroscopy.

The total beta activity was 330 $\mu\text{Ci/g}$. The activity of ^{90}Sr was 108 $\mu\text{Ci/g}$, and the activity of ^{137}Cs was 12.3 $\mu\text{Ci/g}$. To compare these individual measurements with the total beta measurement, the ^{90}Sr was multiplied by 2 to account for the activity of ^{90}Y , which exists in secular equilibrium with ^{90}Sr and would have been counted in the total beta measurement but not counted in the ^{90}Sr measurement. Total beta activity results from the 222-S Laboratory are based on the efficiency of the detector for ^{60}Co . To allow the ^{90}Sr and ^{137}Cs results to be compared with the total beta result, the ^{90}Sr and ^{137}Cs are multiplied by factors of 1.42 and 1.51, respectively, to account for the detector efficiency. This gives a measurement of 325 $\mu\text{Ci/g}$ for the sum of the beta emitters. The ratio of the sum of beta emitters to the total beta measurement is 0.98, indicating good agreement between the two methods. Table B3-1 summarizes the results of the total beta comparison.

Table B3-1. Total Beta Comparison.

Analyte	Concentration ($\mu\text{Ci/g}$)
^{90}Sr	108
^{137}Cs	12.3
Sum of beta emitters ¹	325
Gross beta result	330

Note:

¹The sum of beta emitters was calculated using the equation: $1.42 \times 2 \times ^{90}\text{Sr} + 1.51 \times ^{137}\text{Cs}$. The coefficients 1.42 and 1.51 account for the detector efficiencies for the gross beta, which is calibrated to ^{60}Co . A factor of 2 accounts for the activity of ^{90}Y which exists in secular equilibrium with ^{90}Sr .

Table B3-2 shows the weight percent water results obtained from the TGA and gravimetric methods. The ratio between the samples and duplicates was within 15 percent of 1.0 for all samples except for the core 50, segment 1R, homogenized sample.

Table B3-2. Comparison of Percent Water Results From Thermogravimetric and Gravimetric Analyses. (2 sheets)

Sample	Thermogravimetric Analysis	Gravimetric	Ratio
Core 50, segment 1R, nonhomogenized	5.76	NR	n/a
Core 50, segment 1R, homogenized	26.2	18.0	1.46
Core 50, segment 2, nonhomogenized	29.8	NR	n/a
Core 50, segment 2, homogenized	43.0	41.5	1.04
Core 50, segment 3, nonhomogenized	43.3	IS	n/a
Core 50, segment 4, nonhomogenized	58.1 w/air 0 w/nitrogen	IS	n/a
Core 51, segment 2, homogenized	59.3	60.2	0.99
Core 51, segment 3U, homogenized	59.6	55.1	1.08
Core 51, segment 3L, homogenized	54.2	52.9	1.02
Core 51, segment 4U, homogenized	54.7	55.0	0.99
Core 51, segment 4L, homogenized	53.1	49.5	1.07
Core 52, segment 1, homogenized	15.2	16.7	0.91
Core 52, segment 2, homogenized	55.5	48.5	1.14

Table B3-2. Comparison of Percent Water Results From Thermogravimetric and Gravimetric Analyses. (2 sheets)

Sample	Thermogravimetric Analysis	Gravimetric	Ratio
Core 52, segment 3U, homogenized	54.6	51.4	1.06
Core 52, segment 3L, homogenized	52.2	53.5	0.98
Core 52, segment 4, homogenized	53.5	IS	n/a
Core 50, drainable liquid composite	95.1	95.6	0.99
Core 51, drainable liquid composite	73.7	75.3	0.98
Core 52, drainable liquid composite	82.9	86.5	0.96
Core 51, core solids composite	NR	51.9	n/a
Core 52, core solids composite	NR	47.8	n/a

Notes:

NR = analysis not required

IS = insufficient sample for analysis

B3.3.2 Mass and Charge Balances

The principle objective in performing mass and charge balances is to determine whether the measurements are consistent. Mass and charge balances were only calculated on the solids portion of the waste. In calculating balances, only analytes listed in Table B3-6 with a detected mean of 1,000 $\mu\text{g/g}$ or greater were considered. For the metals, the ICP fusion data were used in all cases. The furnace oxidation results were used for TIC and TOC. Only the core composite means were used.

Table B3-3 lists the cation data. Aluminum was assumed to exist as gibbsite $[\text{Al}(\text{OH})_3]$ because several samples displayed an endothermic reaction during the DSC analysis at 300 °C (572 °F). All phosphorus was assumed to exist as phosphate. Because the ICP phosphorus/IC phosphate comparison revealed that the IC phosphate mean may be biased high, the phosphate values used in the mass and charge balances were calculated from the ICP phosphorus fusion mean, yielding a total of 101,000 $\mu\text{g/g}$ of phosphate. Based on the comparison between the ICP fusion and water digest phosphorus data, it was estimated that approximately 72 percent (or 72,600 $\mu\text{g/g}$) of the phosphate was water soluble. The remaining amount of phosphate, 28,400 $\mu\text{g/g}$, was assumed to be insoluble and exist as the compounds $\text{Ca}_3(\text{PO}_4)_2$, FePO_4 , and BiPO_4 . Based on the silicon data from the core 51 composite, it was determined that silicon exists as the soluble silicate ion. All other cations, except sodium, were assumed to be in their most common hydroxide or oxide form, and the concentrations of the assumed species were calculated stoichiometrically. Because

precipitates are neutral species, all positive charge was attributed to the sodium cation. All TOC was assumed to exist as acetate, and all TIC was assumed to be carbonate. Table B3-4 lists the anions that were assumed to be present as sodium salts and were expected to balance the positive charge exhibited by the cation. The concentrations of the cationic species in Table B3-3, the anionic species in Table B3-4, and the percent water were ultimately used to calculate the mass balance.

The mass balance was calculated from the formula below. The factor 0.0001 is the conversion factor from $\mu\text{g/g}$ to weight percent.

$$\begin{aligned} \text{Mass balance} &= \% \text{ Water} + 0.0001 \times \{\text{Total Analyte Concentration}\} \\ &= \% \text{ Water} + 0.0001 \times \{\text{Al(OH)}_3 + \text{BiPO}_4 + \text{Ca}_3(\text{PO}_4)_2 + \text{FePO}_4 + \\ &\quad \text{FeO(OH)} + \text{Na}^+ + \text{F}^- + \text{NO}_3^- + \text{NO}_2^- + \text{PO}_4^{3-} + \text{SiO}_3^{2-} + \text{SO}_4^{2-} + \\ &\quad \text{CO}_3^{2-} + \text{CH}_3\text{COO}^-\} \end{aligned}$$

The total analyte concentrations calculated from the above equation is 469,000 $\mu\text{g/g}$. The estimated tank weight percent water, based on the gravimetric data, was calculated to be 48.5 weight percent (or 485,000 $\mu\text{g/g}$). Gravimetric results were used in the mass balance because gravimetry gave more representative weight percent water results than the TGA. Adding 485,000 $\mu\text{g/g}$ to the total analyte concentration would produce a mass balance of 95.4 percent, as displayed in Table B3-5. For comparison, a mass balance of 92.9 percent was obtained using TGA data.

The following equations demonstrate the derivation of total cations and total anions; the charge balance is the ratio of these two values.

$$\text{Total cations } (\mu\text{eq/g}) = [\text{Na}^+]/23.0 = 5,350 \mu\text{eq/g}$$

$$\begin{aligned} \text{Total anions } (\mu\text{eq/g}) &= [\text{F}^-]/19.0 + [\text{NO}_3^-]/62.0 + [\text{NO}_2^-]/46.0 + [\text{PO}_4^{3-}]/31.7 + \\ &\quad [\text{SiO}_3^{2-}]/38.0 + [\text{SO}_4^{2-}]/48.0 + [\text{CO}_3^{2-}]/30.0 + \\ &\quad [\text{CH}_3\text{COO}^-]/59.0 = 5,790 \mu\text{eq/g} \end{aligned}$$

The charge balance obtained by dividing the sum of the positive charge by the sum of the negative charge was 0.92.

The above calculations yielded reasonable mass and charge balances (close to 100 percent for mass balance and 1.00 for charge balance), indicating the results are generally consistent. These balances may have been adversely affected by the large variances seen in the results for some analytes. Refer to Section B3.4 for a list of the variances on the mean for each analyte.

Table B3-3. Cation Mass and Charge Data.

Analyte	Concentration ($\mu\text{g/g}$)	Assumed Species	Concentration of Assumed Species ($\mu\text{g/g}$)	Charge ($\mu\text{eq/g}$)
Aluminum	16,400	$\text{Al}(\text{OH})_3$	47,400	0
Bismuth	11,200	BiPO_4	16,300	0
Calcium	1,500	$\text{Ca}_3(\text{PO}_4)_2$	3,870	0
Iron	27,300 ¹	FePO_4	28,200	0
		$\text{FeO}(\text{OH})$	26,800	0
Phosphorus (Phosphorus)	9,200 (44,200) ²	BiPO_4 , $\text{Ca}_3(\text{PO}_4)_2$, FePO_4	--- ³	0
Sodium	123,000	Na^+	123,000	5,350
Total			246,000	5,350

Notes:

¹The extra insoluble phosphorus, which was not attributed to BiPO_4 and $\text{Ca}_3(\text{PO}_4)_2$, was assumed to exist as FePO_4 . The remaining amount of iron was assumed to exist as $\text{FeO}(\text{OH})$.

²The overall ICP fusion mean for phosphorus was 32,900 $\mu\text{g/g}$. Based on a comparison of the fusion and water leach ICP phosphorus data, it was determined that approximately 28 percent of the phosphorus, or 9,200 $\mu\text{g/g}$, were found in an insoluble form. The insoluble portion has been assumed to exist as BiPO_4 , $\text{Ca}_3(\text{PO}_4)_2$, and FePO_4 .

³No value was entered into column four for phosphorus because its mass has already been accounted for in the other compounds.

Table B3-4. Anion Mass and Charge Data.

Analyte	Concentration ($\mu\text{g/g}$)	Assumed Species	Concentration of Assumed Species ($\mu\text{g/g}$)	Charge ($\mu\text{eq/g}$)
Fluoride	11,500	F ⁻	11,500	605
Nitrate	75,400	NO ₃ ⁻	75,400	1,220
Nitrite	11,800	NO ₂ ⁻	11,800	257
Phosphate ¹	72,600 ²	PO ₄ ³⁻	72,600	2,290
Silicon	6,070	SiO ₃ ²⁻	16,500	434
Sulfate	9,970	SO ₄ ²⁻	9,970	208
TIC	4,230	CO ₃ ²⁻	21,200	707
TOC	1,700	CH ₃ COO ⁻	4,180	71
Total			223,000	5,790

Notes:

¹The phosphorus/phosphate comparison in Section B3.3.1 showed a discrepancy between the phosphate values from IC and ICP. The phosphate values derived from the ICP phosphorus data have been used in the mass and charge balances. From the comparison of the ICP fusion and water leach phosphorus data, it was found that approximately 72 percent of the phosphorus was water soluble. Therefore, 72 percent of the total phosphorus value (the ICP fusion phosphorus mean) has been assumed to exist as soluble phosphate, and the concentrations of these fractions have been calculated accordingly.

²Represents total phosphate, and is derived from the ICP fusion phosphorus data as described in footnote 1. This value includes soluble and insoluble phosphate.

Table B3-5. Mass Balance Totals.

Totals	Concentrations ($\mu\text{g/g}$)
Total from Table B3-3	246,000
Total from Table B3-4	223,000
Weight percent water	485,000
Grand total	954,000

B3.3.3 Review of the Analyte Profile

The following conclusions were drawn from review of the available composite and segment analyses.

B.3.3.3.1 Core 50. Interpretation of the results suggested several distinct types of materials. Because of a limited amount of sample and poor sample recovery, no core composite was prepared for core 50, and another core was pulled to compensate for it. The analyses on individual segments revealed high concentrations of aluminum, bismuth, and phosphorus. These results are expected, and the analytes observed generally resemble the composition of CW, TBP, and 1C wastes. The DSC traces for segment 4 show an exotherm beginning around 300 °C. This exothermic region was attributed to a plastic artifact that was mixed with the waste in the last segment of the core. Further analysis of the plastic artifact with different carrier gases showed the artifact to be anomalous and not tank waste. After examining the trends of the segments, all analytes showed a slight drop in concentration toward the middle of core (vertically), except for segment 2, where the core was high in aluminum. This result is in agreement with the high aluminum concentration found in the CW added to the tank late in its service life. The last segment of the core was not recovered; therefore, a conclusion cannot be reached whether the indicator analytes of the 1C waste were present, as found in the other two cores. The change in analyte concentration, as a function of depth of the core observed in the fusion results, is confirmed with the separate acid analysis results from the homogenization tests.

B.3.3.3.2 Core 51. Core 51 contained sufficient sample to prepare a core composite. The trends from the composite indicate high bulk concentrations of bismuth, phosphorus, and aluminum. This behavior is expected from the 1C/CW and CW effluent streams. The solubility of aluminum is lower than expected because of the anticipated presence of $\text{Al}(\text{OH})_3$. The other major constituents that are found in tank 241-T-107 do not behave as expected. Several trace analytes such as boron, lead, lithium, molybdenum, titanium, and thallium produce erratic results with respect to the three different preparation methods. The process history for the waste streams do not indicate the presence of a large amount of these analytes; therefore, the concentrations should be low and variable.

The concentrations of bismuth and phosphorus increase toward the tank bottom. Segment 4L contains the highest concentration of both bismuth and phosphate. These high concentrations are expected at the tank bottom, because the first waste type added to tank 241-T-107 was 1C waste from the early bismuth phosphate process. Although the aluminum concentration at the tank bottom is not the highest observed, the concentration generally increased toward the bottom. This can also be attributed to the 1C waste stream which contained 24 percent aluminum cladding waste. The high concentrations in segment 2 can be attributed to the CW produced from the dissolution of aluminum cladding added late in tank's service life.

The results from anion analysis reveal high overall concentrations of fluoride, nitrate, nitrite, phosphate, and sulfate. The results indicate low concentrations of chloride in the waste. This is not surprising because chloride is not found in any of the waste streams added to tank 241-T-107. The presence of the anions indicates notable quantities of water soluble compounds.

In the assay results related to the safety concerns for TOC and cyanide/ferrocyanide, TOC values for core 51 are low and fall below the established safety criteria of 3 weight percent carbon on a dry weight percent basis. Comparing the results obtained from both laboratories, the 222-S Laboratory results are approximately 3 times higher than those from 325 Analytical Laboratory. Even so, by taking the higher result as a basis, the dry weight organics percent is 0.3 percent, well below the safety criterion. Total cyanide results (from water leach of the sample) for core 51 are higher than for core 52 but well below the safety limits of 3.9 weight percent cyanide.

Analyses for radionuclides were performed on the core composite and segments. The results indicate all radionuclides analyzed by GEA, except ^{137}Cs , are below the detection limit. Cesium-137, prepared by water digestion, produced an average of $9.2 \mu\text{Ci/g}$ of activity, and the fusion digestion produced an average of $12.0 \mu\text{Ci/g}$. In examining the GEA results of the segments, only ^{137}Cs produced any significant amount of activity; the remaining analytes were below the detection limit of the instrument. Strontium concentrations range between 250 and $400 \mu\text{Ci/g}$. Comparisons of the results between water and fusion digestion results indicate mostly soluble cesium and insoluble strontium compounds.

B3.3.3.3 Core 52. The overall high concentrations of aluminum, bismuth, phosphorus, sodium, and silicon agree well with historical records. Aluminum, bismuth, and phosphorus are found in abundance in the waste matrix, and the concentrations strongly indicate 1C and CW wastes. Concentrations for core composite analysis are generally higher for core 52 than for core 51. Aluminum concentrations for core 52 are higher than for core 51 for all three preparation types. A majority of the duplicates for core 52 were not similar and produced high RPDs. The addition of different waste types was observed by the changing concentration throughout the tank depth. By inspecting the concentrations of the analytes by depth, the upper portion of the waste was found to have high concentrations of aluminum attributed to the TBP/CW waste added to the tank late in its service life. The aluminum concentration drops slightly toward the middle of the core, only to increase toward the bottom. The first type of waste added, 1C waste with 24 percent aluminum cladding, could have been responsible for this increase in concentration. Bismuth concentration slowly increased further down into the waste; this was attributed to the 1C waste. Phosphorus concentrations varied as a function of depth for cores 50 and 51 which fits well historically. Core 52, however, does not have the same variations in the tank. This could be caused by the location of the sampling riser with respect to the inlet of the tank.

Examining anion concentrations provides considerable information. High concentrations of fluoride are noted; however, this observation is not surprising because of the presence of ammonium fluoride used in cladding waste and SiF_6^{2-} from 1C waste. Low chloride concentrations are also to be expected because of the lack of this anion in all Hanford waste streams. Nitrate and nitrite were reported in all types of waste and are therefore not considered significant indicator ions (although substantial changes between segments or cores can be suggestive). Cyanide concentrations are low for core 52 (average 45.9 $\mu\text{g/g}$). Examination of the segment analysis results for cyanide reveals segment 2 as containing the highest concentration of cyanide, 0.047 weight percent dry. This concentration of cyanide is extremely low, and is far below the safety limit of 3.9 weight percent.

The first segment of the core had very little water (15.2 percent). The percent of water suggests a formation of a crust. The high concentrations of aluminum seen in the first segments, as well as the DSC scans showing an endothermic region around 100 °C and 300 °C, further suggests the formation of a crust or regional anomaly on top of the waste under riser 3. The total organic carbon analyses indicate low (small) amounts of residual organics in the waste, producing a dry weight percent of 0.38. These two observations affirm the lack of an observable exotherm representative of the tank waste.

The only radionuclides routinely over the detection limit throughout the tank were ^{137}Cs and ^{90}Sr . Cesium-137 concentrations appear to be consistent throughout the tank except for the significant drop in concentration between core 51, segment 2 and segment 3U.

Tank 241-T-107 has slightly lower ^{137}Cs concentrations toward the tank bottom. Comparing the water digestion results with the fusion results indicates most ^{137}Cs is in water soluble forms. Strontium-90 is found primarily in the solids, and its concentration was lower in core 52 than in the other cores. Americium-241 can only be detected in low quantities in segment 1.

B3.3.4 Homogenization Test Description

To comply with Bell (1993), the ability of the process and analytical laboratories to homogenize segments was evaluated. Two homogenization tests were done on samples taken from cores 50, 51, and 52. In the first test, analytical difficulties with the samples were encountered, and the data were not statistically analyzed. In the second test, data from core 51 was incomplete. Consequently, only data from core 50, segment 2, and core 52, segment 3L, were included in the statistical analysis. In the homogenization tests, samples from cores were homogenized and divided into two parts; one subsample was obtained from each part. Two aliquots were taken from each subsample and prepared for chemical analysis. After acid digestion, an ICP analysis was conducted on the samples. For the full discussion including the homogenization test data, refer to Valenzuela and Jensen (1994). It was determined that the process and analytical laboratories were able to homogenize core segments adequately in this experiment.

B3.4 MEAN CONCENTRATIONS AND CONFIDENCE INTERVALS

The statistics in this section were calculated using analytical data from the most recent sampling event of tank 241-T-107. Analysis of variance (ANOVA) techniques were used to estimate the mean and to calculate confidence limits on the mean for all analytes above the detection limit. The estimates of the mean and confidence intervals on the mean were computed based on core composite samples, core segment samples, and drainable liquid samples.

B3.4.1 Mean Concentration Estimates

This section shows three types of analyte concentration estimates: Table B3-6 is based on core composite data, Table B3-7 is based on core segment data, and Table B3-8 is based on results from a chemical analysis of the drainable liquid. The concentration estimates are based on results from ANOVA models fit to the data. These models were fit to the data for all analytes without "less than" values.

The results below are ANOVA estimates based on the core composite data from cores 51 and 52 of tank 241-T-107. In the laboratory, a single core composite sample was formed from the homogenized segment samples of each core. When more than one set of data for the core composite existed, the core means were obtained, then overall means were calculated. Because of incomplete core recovery, the statistical results based on the composite samples are biased. The magnitude of the bias is unknown.

Table B3-6 lists the mean concentrations and the lower limit (LL) and upper limit (UL) to 95 percent confidence intervals on the mean. For some analytes, the LL (95 percent) was negative. Because concentrations are greater than or equal to zero, negative 95 percent LL values were set equal to zero. The ANOVA model used to calculate the summary statistics given in Table B3-6 is outlined in Section B3.4.4.

Table B3-6. Core Composite Statistics (Units are $\mu\text{g/g}$ Except Radionuclides are $\mu\text{Ci/g}$).
(3 sheets)

Analyte	\bar{y}	$s^2(\bar{y})$	df	95% LI	95% UL
ICP.a.Ag	7.37	n/a	1	n/a	n/a
ICP.a.Al	14,400	4.19E+08	1	0.00	2.74E+05
ICP.a.Bi	10,900	3.35E+07	1	0.00	84,400
ICP.a.Ca	723	67,600	1	0.00	4,030
ICP.a.Cd	6.40	2.64	1	0.00	27.0
ICP.a.Cr	354	3,190	1	0.00	1,070
ICP.a.Fe	31,500	1.12E+07	1	0.00	74,000
ICP.a.K	234	342	1	0.00	469
ICP.a.Li	5.48	8.67	1	0.00	42.9
ICP.a.Mg	214	9,410	1	0.00	1,450
ICP.a.Mn	222	361	1	0.00	463
ICP.a.Na	1.30E+05	1.56E+08	1	0.00	2.89E+05
ICP.a.Nd	72.9	5,210	1	0.00	990
ICP.a.Ni	292	625	1	0.00	610
ICP.a.P	30,000	4.90E+05	1	21,100	38,900
ICP.a.Pb	796	3.81E+05	1	0.00	8,640
ICP.a.S	3,050	1.06E+06	1	0.00	16,100
ICP.a.Si	875	2.34E+06	1	0.00	20,300
ICP.a.Sm	285	1.28E+05	1	0.00	4,840
ICP.a.Sr	962	3.09E+05	1	0.00	8,020
ICP.a.Zr	71.6	9,010	1	0.00	1,280
ICP.w.Al	651	1.10E+05	1	0.00	4,860
ICP.w.B	326	3.86E+05	1	0.00	8,220
ICP.w.Bi	243	88,400	1	0.00	4,020
ICP.w.Ca	271	1.68E+05	1	0.00	5,480
ICP.w.Cr	211	600	1	0.00	522
ICP.w.Fe	356	27,900	1	0.00	2,480
ICP.w.K	316	72,900	1	0.00	3,750
ICP.w.Mg	9.83	0.0272	1	7.74	11.9
ICP.w.Mo	7.88	2.66	1	0.00	28.6

Table B3-6. Core Composite Statistics (Units are $\mu\text{g/g}$ Except Radionuclides are $\mu\text{Ci/g}$).
(3 sheets)

Analyte	\bar{y}	$s^2(\bar{y})$	df	95% LL	95% UL
ICP.w.Na	1.08E+05	2.72E+09	1	0.00	7.71E+05
ICP.w.P	23,700	4.32E+07	1	0.00	1.07E+05
ICP.w.S	3,540	7.40E+05	1	0.00	14,500
ICP.w.Sr	5.53	0.00640	1	4.51	6.55
ICP.w.Zr	6.07	56.4	1	0.00	102
ICP.f.Al	16,400	1.14E+08	1	0.00	1.52E+05
ICP.f.Bi	11,200	8.35E+06	1	0.00	47,900
ICP.f.Ca	1,500	1.02E+06	1	0.00	14,300
ICP.f.Cr	347	213	1	161	533
ICP.f.Fe	27,300	9.86E+06	1	0.00	67,300
ICP.f.Mg	232	303	1	11.6	453
ICP.f.Mn	201	502	1	0.00	485
ICP.f.Na	1.23E+05	2.20E+07	1	63,600	1.83E+05
ICP.f.P	32,900	8.50E+05	1	21,200	44,600
ICP.f.Pb	643	6,590	1	0.00	1,670
ICP.f.S	3,140	1.78E+05	1	0.00	8,510
ICP.f.Si	6,070	1.46E+06	1	0.00	21,400
ICP.f.Sr	861	8,650	1	0.00	2,040
ICP.f.Zr	113.0	1,370	1	0.00	583
Total dissolved solids (wt%)	0.395	5.63E-05	1	0.299	0.490
RS (wt%)	28.7	30.8	1	0.00	99.2
CN	68.8	2,100	1	0.00	651
IC.F	11,500	4.95E+06	1	0.00	39,700
IC.Cl	547	18,300	1	0.00	2,270
IC.NO ₂ ⁻	11,800	1.21E+07	1	0.00	55,900
IC.NO ₃ ⁻	75,400	3.03E+08	1	0.00	2.97E+05
IC.PO ₄ ³⁻	1.14E+05	3.80E+08	1	0.00	3.62E+05
IC.SO ₄ ²⁻	9,970	7.16E+06	1	0.00	44,000
Spec.w.NO ₂ ⁻	11,100	3.94E+07	1	0.00	90,800
Furnace ox. TIC	4,230	8.41E+06	1	0.00	41,100

Table B3-6. Core Composite Statistics (Units are $\mu\text{g/g}$ Except Radionuclides are $\mu\text{Ci/g}$).
(3 sheets)

Analyte	\bar{y}	$s^2(\bar{y})$	df	95% LL	95% UL
Persulfate ox. TIC	1,700	1.44E+05	1	0.00	6,530
Furnace ox. TOC	1,700	6,890	1	0.00	5,030
Persulfate ox. TOC	360	1,600	1	0.00	868
Total alpha (fusion)	0.434	0.00152	1	0.00	0.930
Total alpha (water)	0.00398	1.19E-05	1	0.00	0.0479
Am-241 (fusion)	0.0140	7.49E-06	1	0.00	0.0488
Pu-239/40 (fusion)	0.150	0.0011	1	0.00	0.572
Total beta (fusion)	330	5,370	1	0.00	1,260
Total beta (water)	13.4	9.49	1	0.00	52.6
Sr-90 (fusion)	108	530	1	0.00	401
Tc-99 (fusion)	0.0505	5.06E-06	1	0.0219	0.0791
GEA.Cs-137 (fusion)	12.3	2.56	1	0.00	32.6
RS.GEA.Cs-137 (water)	10.6	16.7	1	0.00	62.6
U (LF)	22,600	1.35E+07	1	0.00	69,300
U-238 ¹	0.00831	2.86E-06	1	0.00	0.0298
% Solids	51.5	11.3	1	8.77	94.2
pH	11.5	0.0400	1	8.96	14.0
CN.dir	76.1	1,560	1	0.00	577

Notes: n/a = not applicable
df = degrees of freedom
ox. = oxidation
RS = residual solids from water digestion.

¹Calculated from the uranium (fusion) laser fluorimetry data.

Inventory estimates can be calculated for each analyte using an average density of 1.51 g/mL and a waste volume of 655 kL (173 kgal). The kg estimates are the concentration estimates given in Table B3-6 multiplied by $1.51 \times 655 / 1000$. The Ci estimates are the concentration estimates in Table B3-6 multiplied by 1.51×655 .

B3.4.2 Segment-Level Means

Segment data were analyzed for all three tank 241-T-107 core samples. The degrees of freedom used in calculating the confidence intervals is the number of core samples minus one. The confidence intervals computed for the segment level means are based on two degrees of freedom. The confidence intervals in the previous section were based on one degree of freedom.

Table B3-7 contains the summary statistics, by analyte, for ICP KOH/Ni fusion dissolution, radiochemistry, and other analyses. These values are based on a chemical analysis of segments from each core. The ANOVA model used to determine the summary statistics in Table B3-7 is described in Section B3.4.5.

Inventory estimates can be calculated for each analyte using an average density of 1.51 g/mL and a waste volume of 655 kL (173 kgal). The kg estimates are the concentration estimates in Table B3-7 multiplied by 1.51*655/1000. The Ci estimates are the concentration estimates in Table B3-7 multiplied by 1.51*655.

Table B3-7. Segment Data Concentration Estimate Statistics
(Units $\mu\text{g/g}$ Except Radionuclides $\mu\text{Ci/g}$). (2 sheets)

Analyte	\bar{y}	$s^2(\bar{y})$	df	95% LL	95% UL
ICP.f.Al	38,800	4.34E+08	2	0.00	1.28E+05
ICP.f.Bi	9,010	5.71E+06	2	0.00	19,300
ICP.f.Ca	1,920	7.16E+05	2	0.00	5,560
ICP.f.Cd	9.87	6.79	2	0.00	21.1
ICP.f.Cr	317	1,150	2	171	463
ICP.f.Fe	26,000	7.86E+06	2	13,900	38,000
ICP.f.Mg	261	1,250	2	109	412
ICP.f.Mn	427	42,700	2	0.00	1,320
ICP.f.Na	95,500	9.06E+07	2	54,600	1.36E+05
ICP.f.Ni	5,190	1.59E+06	2	0.00	10,600
ICP.f.P	21,300	1.80E+07	2	3,040	39,600
ICP.f.Pb	717	29,600	2	0.00	1,460
ICP.f.S	2,910	2.55E+05	2	736	5,080
ICP.f.Si	5,390	1.05E+06	2	982	9,800
ICP.f.Sr	793	34,700	2	0.00	1,590
ICP.f.Ti	46.8	592	2	0.00	151

Table B3-7. Segment Data Concentration Estimate Statistics
(Units $\mu\text{g/g}$ Except Radionuclides $\mu\text{Ci/g}$). (2 sheets)

Analyte	\bar{y}	$s^2(\bar{y})$	df	95% LL	95% UL
ICP.f.Zr	55.2	1,090	2	0.00	197
Persulfate ox. TIC	2,460	56,800	2	1,440	3,490
Persulfate ox. TOC	868	41,300	2	0.00	1,740
Sr-90	119	959	2	0.00	252
GEA.Cs-137	20.5	140.0	2	0.00	71.5
TGA.H ₂ O	46.0	30.7	2	22.2	69.8
% Solids	58.0	47.0	2	27.9	87.5
pH	11.1	0.0194	2	10.5	11.7
CN	63.3	202	2	2.24	124

Table B3-8 contains the summary statistics, by analyte, for ICP, IC, radiochemistry, and several special analyses of acidified drainable liquid samples. These values are based on a chemical analysis of a drainable liquid sample from each of three cores.

B3.4.3 Drainable Liquid Means

Drainable liquid sample data was available from each of three core samples taken from tank 241-T-107. The ANOVA model used to describe the data is identical to the model used for the core composite data. The confidence intervals computed from the drainable liquid data were based on two degrees of freedom. Because of incomplete core segment recovery, the results in Table B3-8 may be biased. The magnitude of the bias is unknown.

Table B3-8. Drainable Liquid Composite Data Concentration Estimate Statistics
(Units $\mu\text{g/mL}$ Except Radionuclides $\mu\text{Ci/mL}$). (2 sheets)

Analyte	\bar{y}	$s^2(\bar{y})$	df	95% LL	95% UL
ICP.Al	21.2	182	2	0.00	79.2
ICP.B	21.3	39.4	2	0.00	48.3
ICP.Ca	4.40	0.0926	2	3.09	5.71
ICP.Cr	169	4,410	2	0.00	455
ICP.Fe	24.3	156	2	0.00	77.9
ICP.K	184	9,080	2	0.00	594

Table B3-8. Drainable Liquid Composite Data Concentration Estimate Statistics
(Units $\mu\text{g/mL}$ Except Radionuclides $\mu\text{Ci/mL}$). (2 sheets)

Analyte	\bar{y}	$s^2(\bar{y})$	df	95% LL	95% UL
ICP.Mg	0.681	0.0781	2	0.00	1.88
ICP.Mo	8.90	45.5	2	0.00	37.9
ICP.Na	54,000	5.47E+08	2	0.00	1.55E+05
ICP.Ni	6.87	21.2	2	0.00	26.7
ICP.P	1,800	2.82E+05	2	0.00	4,090
ICP.S	3,390	2.24E+06	2	0.00	9,830
ICP.Si	70.7	73.1	2	33.9	107
ICP.Sr	0.707	0.304	2	0.00	3.08
ICP.Zr	0.826	0.211	2	0.00	2.80
CN	68.4	1,800	2	0.00	251
IC.F	557	38,700	2	0.00	1,400
IC.Cl	799	1.10E+05	2	0.00	2,230
IC.NO ₂ ⁻	12,800	5.79E+07	2	0.00	45,500
IC.PO ₄ ³⁻	5,420	2.45E+06	2	0.00	12,200
IC.SO ₄ ²⁻	10,300	1.25E+07	2	0.00	25,500
IC.NO ₃ ⁻	85,200	1.12E+09	2	0.00	2.30E+05
Spec.w.NO ₂ ⁻	7,570	8.04E+06	2	0.00	19,800
NH ₃	115	2,940	2	0.00	348
TIC	1,800	1.89E+06	2	0.00	7,710
TOC	857	63,600	2	0.00	1,940
Total alpha	0.00601	2.81E-05	2	0.00	0.0288
U (LF)	241	30,300	2	0.00	990
U-238 ¹	8.10E-05	3.42E-09	2	0.00	3.33E-04
Am-241	1.15E-04	1.99E-09	1	0.00	6.82E-04
Total beta	11.8	47.4	2	0.00	41.4
Sr-90	0.0594	0.00109	2	0.00	0.202
Tc-99	0.0368	6.06E-04	2	0.00	0.143
C-14	2.03E-04	2.50E-08	2	0.00	8.84E-04
H-3	0.00180	2.01E-06	2	0.00	0.00803
GEA.Cs-137	8.43	25.6	2	0.00	30.2

Note:

¹Calculated from the uranium (fusion) laser fluorimetry data.

Inventory estimates can be calculated for each analyte using the drainable interstitial liquid volume of 83 kL (22 kgal). The kg estimates are the concentration estimates in Table B3-8 multiplied by 83/1000. The Ci estimates are the concentration estimates in Table B3-8 multiplied by 83.

B3.4.4 ANOVA Models for Core Composite and Drainable Liquid Data

The statistical model that describes the structure of the core composite data and drainable liquid data is

$$y_{ij} = \mu + C_i + A_{ij}, \quad i = 1, \dots, a, \quad j = 1, \dots, n_i,$$

where

- y_{ij} = laboratory results from the j^{th} duplicate of the i^{th} core composite sample from the tank
- μ = the grand mean
- C_i = the effect of the i^{th} core (spatial effect)
- A_{ij} = the analytical error associated with the j^{th} duplicate from the i^{th} core
- a = the number of cores
- n_i = the number of analytical results from the i^{th} core.

This is an unbalanced one-way random effects analysis of variance model. The C_i variable is assumed to be a random effect. It is assumed that C_i and A_{ij} are each distributed normally with mean zero and variances of $\sigma^2(C)$ and $\sigma^2(A)$, respectively. Estimates of $\sigma^2(C)$ and $\sigma^2(A)$ were obtained using Restricted Maximum Likelihood Estimation (REML). This method, as applied to variance component estimation, is described by Harville (1977). The REML estimates were obtained using the statistics program S-PLUS⁵ (Statistical Sciences 1993).

⁵S-PLUS is a registered trademark of Statistical Sciences, Seattle, Washington.

An estimate of the true unknown mean concentration μ is the mean of the core means; that is, each core is weighted equally.

$$\hat{\mu} = \bar{y} = \frac{1}{a} \sum_{i=1}^a \bar{y}_{i\cdot}, \text{ where } \bar{y}_{i\cdot} = \sum_{j=1}^{n_i} \frac{y_{ij}}{n_i}.$$

The variance of \bar{y} (Snedecor and Cochran 1980) is

$$\sigma^2(\bar{y}) = \frac{\sigma^2(C)}{a} + \frac{\sigma^2(A)}{a^2} \sum_{i=1}^a \frac{1}{n_i}.$$

The estimated variance of \bar{y} is obtained by substituting the REML estimators of the two variance components into the above equation. The degrees of freedom associated with the estimate of variance is the number of cores with data minus one.

The 95 percent confidence interval (LL and UL) on the mean concentration μ are

$$95\% \text{ LL} = \bar{y} - t_{0.025} \sqrt{\hat{\sigma}^2(\bar{y})}, \text{ and } 95\% \text{ UL} = \bar{y} + t_{0.025} \sqrt{\hat{\sigma}^2(\bar{y})},$$

where $t_{0.025}$ is the 0.025 quantile from a Student's t-distribution with a-1 degree of freedom. In this case, the number of composite samples (a) equals 2, so that the estimates are based on one degree of freedom and $t_{0.025} = 12.706$.

B3.4.5 ANOVA Model for Core Segment Data

The statistical model that describes the structure of the core segment data is

$$y_{ijk} = \mu + C_i + S_{ij} + A_{ijk},$$

$$i = 1, \dots, a, \quad j = 1, \dots, b_i, \quad k = 1, \dots, n_{ij},$$

where

- y_{ijk} = laboratory results from the k^{th} duplicate of the j^{th} segment of the i^{th} core from the tank
- μ = the grand mean
- C_i = the effect of the i^{th} core (spatial effect)

- S_{ij} = the effect of the j^{th} segment sample from the i^{th} core (spatial effect)
- A_{ijk} = the analytical error associated with the k^{th} duplicate in the j^{th} composite from the i^{th} core
- a = the number of cores
- b_i = the number of segments (half segments) in the i^{th} core
- n_{ij} = the number of analytical results from the j^{th} segments (half segments) in the i^{th} core.

The number of segments (half segments) in cores 50, 51, and 52 were 3, 5, and 4 (that is, $b_1 = 3$, $b_2 = 5$ and $b_3 = 4$).

The variables C_i and S_{ij} are treated as random effects. It is assumed that C_i , S_{ij} , and A_{ijk} are each distributed normally with mean zero and variances of $\sigma^2(C)$, $\sigma^2(S)$, and $\sigma^2(A)$, respectively. Estimates of $\sigma^2(C)$, $\sigma^2(S)$, and $\sigma^2(A)$ were obtained using REML methods. The REML estimation method, as applied to variance component, is described by Harville (1977). The REML estimates were obtained using the statistics program S-PLUS® (Statistical Sciences 1993).

The mean concentration of each analyte of interest in the tank was calculated using the following equation:

$$\bar{y} = \frac{1}{a} \sum_{i=1}^a \bar{y}_{i++} = \frac{1}{a} \sum_{i=1}^a \frac{\sum_{j=1}^{b_i} \sum_{k=1}^{n_{ij}} y_{ijk}}{n_{i+}} = \frac{1}{a} \sum_{i=1}^a \frac{\sum_{j=1}^{b_i} \sum_{k=1}^{n_{ij}} (\mu + C_i + S_{ij} + A_{ijk})}{n_{i+}},$$

where

$$\bar{y}_{i++} = \frac{\sum_{j=1}^{b_i} \sum_{k=1}^{n_{ij}} y_{ijk}}{n_{i+}} \quad \text{and} \quad n_{i+} = \sum_{j=1}^{b_i} n_{ij}.$$

This mean equally weights the results from each core regardless of the unbalance that may exist for a particular analyte.

The variance of \bar{y} is

$$V(\bar{y}) = C_1\sigma^2(C) + C_2\sigma^2(S) + C_3\sigma^2(A)$$

where

$$C_1 = \frac{1}{a}, \quad C_2 = \frac{1}{a^2} \sum_{i=1}^a \left(\frac{1}{n_{i+}} \right)^2 \left(\sum_{j=1}^{b_i} n_{ij}^2 \right), \quad C_3 = \frac{1}{a^2} \sum_{i=1}^a \left(\frac{1}{n_{i+}} \right).$$

Using $\hat{\sigma}^2(C)$, $\hat{\sigma}^2(S)$, and $\hat{\sigma}^2(A)$ (REML variance component estimates), an estimated variance of \bar{y} is

$$\hat{\sigma}^2(\bar{y}) = C_1\hat{\sigma}^2(C) + C_2\hat{\sigma}^2(S) + C_3\hat{\sigma}^2(A).$$

The approximate degrees of freedom used for $\hat{\sigma}^2(\bar{y})$ is the number of cores with data minus one.

The lower and upper 95 percent limits (95 percent LL and 95 percent UL, respectively) on the mean concentration are

$$95\% \text{ LL} = \bar{y} - t_{0.025} \sqrt{\hat{\sigma}^2(\bar{y})} \quad \text{and} \quad 95\% \text{ UL} = \bar{y} + t_{0.025} \sqrt{\hat{\sigma}^2(\bar{y})}$$

where $t_{0.025}$ is the 0.025 quantile from a Student's t-distribution with approximate degrees of freedom equal to the number of cores with data minus one. In this case, the degrees of freedom is two and $t_{0.025} = 4.303$.

B4.0 APPENDIX B REFERENCES

- Bell, K. E., 1993, *Tank Waste Remediation System Tank Waste Characterization Plan*, WHC-SD-WM-PLN-047, Rev. 1, Westinghouse Hanford Company, Richland, Washington.
- Bratzel, D. R., 1985, *Tank 107-T Waste Mixing Study*, (internal letter 65453-85-043 to L. A. Gale, March 5), Rockwell Hanford Company Richland, Washington.
- Bratzel, D. R., 1989, *Interim Results of T-101 and T-107 Tank 107-T Analyses and T-101/Neutralized Plutonium Finishing Plant Acid Waste*, (internal memorandum 12712-PCL89-144 to A. J. DiLiberto, August 1), Rockwell Hanford Company Richland, Washington.
- Burnum, S. T., 1995, *Qualification of Reported WHC Program Data*, (letter 95-CHD-065 to president, Westinghouse Hanford Company, August 18), U.S. Department of Energy, Richland Operations Office, Richland, Washington.
- Carpenter, B. C., 1995, *Tank 241-T-107 Tank Characterization Plan*, WHC-SD-WM-TP-295, Rev. 0A, Westinghouse Hanford Company, Richland, Washington.
- De Lorenzo, D. S., J. H. Rutherford, D. J. Smith, D. B. Hiller, K. W. Johnson, and B. C. Simpson, 1994, *Tank Characterization Reference Guide*, WHC-SD-WM-TI-648, Rev. 0, Westinghouse Hanford Company, Richland, Washington.
- Dukelow, G. T., J. W. Hunt, H. Babad, and J. E Meacham, 1995, *Tank Safety Screening Data Quality Objective*, WHC-SD-WM-SP-004, Rev. 2, Westinghouse Hanford Company, Richland, Washington.
- Godfrey, W. L., 1965, *242-T Evaporator Feed*, (internal letter to S. J. Beard, September 24), General Electric, Richland, Washington.
- Harville, D. A., 1977, "Maximum Likelihood Approaches to Variance Component Estimation and to Related Problems," *Journal of the American Statistical Association*, pp. 324-340, Washington, D.C.
- Huckaby and Bratzel, D. R., 1995, *Tank 241-T-107 Headspace Gas and Vapor Characterization Results for Samples Collected in January 1995*, WHC-SD-WM-ER-447, Rev. 0, Westinghouse Hanford Company, Richland, Washington.

- Osborne, J. W., J. L. Huckaby, T. P. Rudolph, E. R. Hewitt, D. D. Mahlum, J. Y. Young and C. M. Anderson, 1994, *Data Quality Objectives for Generic In-Tank Health and Safety Issue Resolution*, WHC-SD-WM-DQO-002, Rev. 0, Westinghouse Hanford Company, Richland, Washington.
- Osborne, J. W., and L. L. Buckley, 1995, *Data Quality Objectives for Tank Hazardous Vapor Safety Screening*, WHC-SD-WM-DQO-002, Rev. 2, Westinghouse Hanford Company, Richland, Washington.
- Pingel, L. A., 1992, *Waste Tank T-107 Vapor Sampling Results*, (internal memorandum 12240-SAS93-003 to G. L. Dukelow, November 2), Westinghouse Hanford Company, Richland, Washington.
- Pool, K. H., R. B. Lucke, B. D McVeety, G. S Klinger, T. W. Clauss, M. W. Ligojke, K. B. Olsen, O. P. Bredt, J. S. Fruchter, and S. C. Goheen, 1995, *Vapor Space Characterization of Waste Tank 241-T-107: Results from Samples Collected on 1/18/95*, PNL-10595, Pacific Northwest National Laboratory, Richland, Washington.
- Silvers, K. L., and A. F. Noonan, 1993, *Letter of Instruction for Third Core Sample From Tank T-107*, (internal letter 7K2220-93-012 to J. G. Kristofzski, February 19), Westinghouse Hanford Company, Richland, Washington.
- Snedecor, G. W., and W. G. Cochran, 1980, *Statistical Methods*, 7th Edition, Iowa State University Press, Ames, Iowa.
- Statistical Sciences, 1993, *S-PLUS Reference Manual Version 3.2*, StatSci, a division of MathSoft, Inc., Seattle, Washington.
- Svancara, G. B., and K. N. Pool, 1993, *WHC 222-S and PNL-325 Single-Shell Tank Waste Characterization, 241-T-107 Cores 50, 51, and 52 - Data Package and Validation Summaries*, WHC-SD-WM-DP-042, Rev. 1A, Westinghouse Hanford Company, Richland, Washington.
- Valenzuela, B. D., and L. Jensen, 1994, *Tank Characterization Report for Single-Shell Tank 241-T-107*, WHC-SD-WM-ER-382, Rev. 0A, Westinghouse Hanford Company, Richland, Washington.
- Wheeler, R. E., 1975, *Analysis of Tank Farm Samples, Sample: T-8248, Tank: 107-T, Received: September 25, 1975*, (internal letter to R. L. Walser, November 26), Atlantic Richfield Hanford Company, Richland, Washington.
- Winters, W. I., J. G. Hill, B. C. Simpson, J. W. Buck, P. J. Chamberlain, and V. L. Hunter, 1991, *Waste Characterization Plan for the Hanford Site Single-Shell Tanks*, WHC-EP-0210, Rev. 3, Westinghouse Hanford Company, Richland, Washington.
-
-

This page intentionally left blank.

APPENDIX C

STATISTICAL ANALYSIS FOR ISSUE RESOLUTION

This page intentionally left blank.

APPENDIX C**STATISTICAL ANALYSIS FOR ISSUE RESOLUTION**

In Appendix C, the data investigations required for the applicable DQOs for tank 241-T-107 would normally be reported. These include statistical and other numerical manipulations required in the DQOs. Because the 1992 and 1993 core sampling of tank 241-T-107 predated DQOs, none were applicable to the sampling event. Although an effort has been made to apply the current safety screening DQO requirements to the 1992 and 1993 data set, no computations were made as required by the DQO. Specifically, no confidence intervals were calculated for the total alpha data for each sample/duplicate pair. These calculations were not done because all total alpha results were quite low, and total alpha was only measured on the core composites instead of each half segment as required by the DQO. A 95 percent confidence interval was calculated on the overall tank mean, yielding a UL of 0.930 $\mu\text{Ci/g}$. No confidence intervals were necessary for the DSC results because no exotherms were found in any waste samples (the lone exotherm was attributed to a plastic scrap which had been commingled with the sample).

The vapor DQO (Osborne et al. 1994) was applicable to the 1995 vapor sampling event. However, the DQO does not require statistical calculations for issue resolution.

C1.0 APPENDIX C REFERENCES

Osborne, J. W., J. L. Huckaby, T. P. Rudolph, E. R. Hewitt, D. D. Mahlum, J. Y. Young and C. M. Anderson, 1994, *Data Quality Objectives for Generic In-Tank Health and Safety Issue Resolution*, WHC-SD-WM-DQO-002, Rev. 0, Westinghouse Hanford Company, Richland, Washington.

This page intentionally left blank.

APPENDIX D

**EVALUATION TO ESTABLISH BEST-BASIS INVENTORY FOR
SINGLE-SHELL TANK 241-T-107**

This page intentionally left blank.

APPENDIX D

EVALUATION TO ESTABLISH BEST-BASIS INVENTORY FOR SINGLE-SHELL TANK 241-T-107

An effort is underway to provide waste inventory estimates that will serve as standard characterization source terms for various waste management activities (Hodgson and LeClair 1996). As part of this effort, an evaluation of available waste related information for tank 241-T-107 was performed, and a best-basis inventory was established. This work follows the methodology that was established by the standard inventory task.

D1.0 IDENTIFY/COMPILE INVENTORY SOURCES

Appendix B provides the characterization results from the most recent sampling event. Section B3.4 provides mean concentrations calculated from the analytical results. Three push-mode core samples were obtained from three risers in November 1992 to March 1993. Samples from each core and core composites from two cores were analyzed.

Component inventories can be calculated by multiplying the concentration of an analyte by the current tank volume and by the density of the waste. The HDW model document (Agnew et al. 1996a) provides tank content estimates, derived from the Los Alamos National Laboratory model, in terms of component concentrations and inventories. Appendix D lists the data sources used in this evaluation.

D2.0 COMPARE COMPONENT INVENTORY VALUES

The sample-based inventory, derived from analytical mean concentration data for the most recent sampling event (see Appendix B), and model-based inventory, generated by the HDW model (Agnew et al. 1996a), are compared in Tables D2-1 and D2-2. Table D2-1 compares nonradioactive components on a kilogram basis, and Table D2-2 compares the radioactive components on a curie basis.

The sample-based inventory listed in Tables D2-1 and D2-2 was calculated by multiplying each mean analyte concentration value by the current tank volume, 655 kL (173 kgal) (Hanlon 1996), and by the mean density of the waste, 1.51 g/mL (see Appendix B). At the time the tank was sampled, the waste volume was reported as 681 kL (180 kgal) with 647 kL (171 kgal) of sludge and 34 kL (9 kgal) of supernatant. The HDW model-based inventory was derived using this volume and 1.28 g/mL as the mean density.

Table D2-1. Sampling-Based and Hanford Defined Waste-Based Inventory Estimates for Nonradioactive Components in Tank 241-T-107.

Analyte	Sampling ¹ Inventory Estimate (kg)	HDW ² Inventory Estimate (kg)	Analyte	Sampling ¹ Inventory Estimate (kg)	HDW ² Inventory Estimate (kg)
Al	16,200	8,300	Ni	289	44.1
Bi	11,100	10,400	NO ₂	11,700	5,330
Ca	1,480	1,960	NO ₃	74,600	15,000
Cd	6.3	nr	OH	nr	28,800
Cl	541	340	P as PO ₄	1.128E+05	81,900
Cr	343	127	Si as SiO ₃	6,000	1,150
F ³	11,400	2,030	S as SO ₄	9,860	2,780
Fe	27,000	11,700	Sr	852	nr
FeCN/CN	68.8	nr	TIC	4,180	2,930
Hg	nr	13.7	TOC	1,680	0
K	231	81.7	U _{TOTAL}	22,400	88
Mg	229	nr	Zr	112	608
Mn	199	nr	H ₂ O (wt%)	4.55E+05 (46)	5.963E+05
Na	1.217E+05	72,600	Density (g/mL)	1.51	1.28
NH ₃	nr	1.97			

Notes:

nr = not reported

¹Appendix B²Agnew et al. (1996a)³Fluoride based on water soluble portion only.

Table D2-2. Sampling and Hanford Defined Waste Predicted Inventory Estimates for Radioactive Components in Tank 241-T-107.

Analyte	Sampling ¹ Inventory Estimate (Ci)	HDW ² Inventory Estimate (Ci)	Analyte	Sampling ¹ Inventory Estimate (Ci)	HDW ² Inventory Estimate (Ci)
⁹⁰ Sr	1.068E+05	69.8	²⁴¹ Am	13.8	nr
⁹⁹ Tc	50	nr	Total α	429	nr
¹³⁷ Cs	12,200	7,890	Total β	3.264E+05	nr

Notes:

nr = not reported

¹Appendix B²Agnew et al. (1996a)**D3.0 REVIEW AND EVALUATION OF COMPONENT INVENTORIES**

The following evaluation of tank contents is performed to identify potential errors and/or missing information that would influence the sampling-based and HDW model component inventories. Computations for the HDW model are based on the assumption that most solid waste in tank 241-T-107 is a single waste type, 1C waste; the remaining unknown waste has also been designated as 1C. The implication is that all tank waste is a single type. Both the waste transfer history and the sampling results indicate that other waste types have contributed to the solids in tank 241-T-107. Appendix A includes a complete summary of the waste transfer history of tank 241-T-107. An abbreviated summary follows and highlights the waste types added to the tank that may have an effect on the solid waste inventory.

Tank 241-T-107 is the first tank in a cascade that includes tanks 241-T-108 and 241-T-109. The tank was filled in 1945 and 1946 with 1C waste and was undisturbed until 1951 when over half the supernatant was removed. The tank then received TBP waste, also known as uranium recovery (UR) waste, until mid-1953. The supernatant was removed again and replaced with unconcentrated, ferrocyanide-scavenged TBP waste. Between 1954 and late 1966, no waste transfers into or out of tank 241-T-107 were recorded, and the waste volume remained constant.

Tank 241-T-107 received cladding waste in 1967. Both aluminum and Zircaloy CW are expected in the tank. In 1969, approximately half the waste content, mostly liquids with a small amount of solids, was removed from tank 241-T-107. In 1973, the tank received ion

exchange (IX) waste, which was immediately distributed to other tanks. In 1976, tank 241-T-107 was removed from service and designated inactive.

D3.1 CONTRIBUTING WASTE TYPES

The waste types expected to have accumulated in tank 241-T-107, as reported by various sources, are listed in Table D3-1. The waste transfer history (see Appendix A) reveals that several waste types were added to tank 241-T-107. The HDW model (Agnew et al. 1996a) assumes these other waste types do not contribute to the tank inventory significantly at this time. Agnew et al. (1996a) predicts the presence of 496 kL (131 kgal) of 1C1 waste, and an additional 150 kL (40 kgal). The calculations that follow indicate constituents of other waste types are present.

Table D3-1. Expected Solids for Tank 241-T-107.

Reference	Waste Type
HDW Model (Agnew et al. 1996a)	1C1
SORWT (Hill et al. 1995)	1C, CW, UR
WSTRS (Agnew et al. 1996b) (see Appendix A)	1C1, CW, UR, IX

Notes:

- 1C = First cycle decontamination BiPO₄ waste
- 1C1 = First cycle decontamination BiPO₄ waste, specifically before 1950
- CW = Cladding waste produced at PUREX from dissolution of aluminum and/or Zircaloy fuel cladding
- IX = Ion exchange waste from the cesium recovery process at B Plant
- SORWT = Sort of Radioactive Waste Types (model)
- UR = Uranium recovery waste from uranium recovery operations; also called TBP waste

D3.2 EVALUATION OF TECHNICAL INFORMATION

Table D3-2 compares technical flowsheet information for the 1C (Kupfer 1996) and TBP (Hill et al. 1995) waste streams and the corresponding HDW model waste streams (Agnew et al. 1996a). According to the HDW model, cladding wastes were sent to the same tanks as first decontamination cycle waste until 1954. The model assumes a mixture of the two waste types, with cladding waste making up approximately 24 percent of the waste stream. The HDW also divides the pre-1950 1C waste from the waste transferred to the tanks after 1950, designating them as 1C1 and 1C2, respectively. The tank was filled in 1946 and was undisturbed until 1951; therefore, only the 1C1 waste type is predicted to be in the tank. According to the transfer history, the tank did not receive additional 1C waste after 1951.

Table D3-2. Technical Flowsheet and Hanford Defined Waste Streams.

Analyte	Flowsheet ¹ 1C/CW (M)	Flowsheet ² TBP (M)	HDW ³ 1C/CW (M)	HDW ³ TBP (M)
Al	0.0826	nr	0.233	nr
Bi	0.0115	nr	0.014	nr
Cr	0.00306	nr	0.0052	0.0032
F	0.17	nr	0.228	nr
Fe	0.0315	0.03	0.046	0.046
Na	2.17	8.87	2.24	4.50678
Si	0.0312	nr	0.038	0.1416
U	0.000963	0.0061	0.000767	0.0078
Zr	0.000296	nr	0.004	nr
NO ₂	0.0577	nr	0.174	nr
NO ₃	1.44	7.35	0.588	3.40208
PO ₄	0.258	0.3	0.326	0.13
SO ₄	0.0631	0.31	0.0616	0.1416

Notes:

nr = not reported

¹Appendix C of Kupfer (1996)²Hill et al. (1995)³Appendix B of Agnew et al. (1996)

The waste transfer history indicates that several types of waste were added to tank 241-T-107 over time. Analytical results support this history. The analytical core segment results reveal the tank is horizontally homogeneous and vertically heterogeneous (Jensen et al. 1994).

The history indicates that in addition to the 1C/CW waste noted by Agnew et al. (1996a), tank 241-T-107 received TBP and aluminum and/or Zircaloy CW. The history also indicates IX waste was added to the tank. This waste was mostly liquid and probably mixed with the existing supernatant; it does not appear to have had an effect on the solids. Much of the supernatant was transferred from the tank shortly after the IX waste was added.

D3.3 ASSUMPTIONS FOR RECONCILING WASTE INVENTORIES

Inventories of certain components in tank 241-T-107 were estimated using an engineering assessment that is based on a set of simplified assumptions. The inventories were compared with tank 241-T-107 sample-based inventories and HDW model inventories.

The assumptions and observations for the engineering assessment were based on best technical judgment pertaining to parameters that can significantly influence tank inventories. These parameters include the following:

1. contributing waste types and correct relative proportions of the waste types
2. model flowsheet conditions, fuel processed, and waste volumes
3. partitioning of components
4. physical parameters such as density, percent solids, and void fraction.

The assumptions can be modified to provide a basis for identifying potential errors that could influence the sampling-based and model-based inventories. The following simplified assumptions and observations were used for the assessment.

1. Analytical data from tanks 241-T-104 and 241-BX-112, which contain only 1C waste, and tank 241-TY-105, which contains only TBP waste, helped provide the analytical basis for estimating the inventory in tank 241-T-107.
2. The waste in tank 241-T-107 consists of 1C waste from the BiPO₄ process and the associated CW waste, uranium recovery waste from the TBP process, and aluminum and/or Zircaloy cladding waste. Ion exchange waste made no significant contributions to the solids in the tank.
3. Components listed in the technical flowsheets (see Table D3-1) were used for the evaluation. Cerium is not expected in significant amounts and was not reported in the sampling data. This evaluation makes no estimate about the effect of the cladding waste which was added late in the tank transfer history.
4. Tank inventory comparisons are made on a same volume basis, using the tank volume listed in Hanlon (1996). The engineering evaluation, the sampling-based inventories, and the HDW model-based inventories use equivalent volumes of 655 kL (173 kgal) solids.
5. The waste in the tank is treated as two distinct layers. The bottom layer is composed of 496 kL (131 kgal) of 1C waste; the upper layer 159 kL (42 kgal) of TBP waste. The constituents of these wastes are assumed to be evenly distributed throughout their respective layers.

-
-
6. All Bi, Si, and Zr are assumed to precipitate as water insoluble components in the 1C waste. The Fe and U are assumed to precipitate in 1C and TBP wastes. Bi, Si, and Zr are not expected in TBP waste.
 7. The Al, Cr, Na, F, NO₂, and PO₄ are assumed to partition between the liquid and solid phases. Al, Cr, F, and NO₂ are not assigned to TBP waste.
 8. The NO₃ and SO₄ are assumed to remain dissolved in the interstitial liquid in 1C and TBP waste.
 9. No radiolysis of NO₃ to NO₂ and no additions of NO₂ to the waste for corrosion purposes are factored into this assessment.

D3.4 CONCENTRATION FACTOR AND PARTITIONING FACTORS FOR TANK 241-T-107

One method for estimating a component inventory for a particular waste type in a tank is to derive a concentration factor (CF) for that component. This approach was used to estimate inventories in tank 241-T-107. Concentration factors are a method of reconciling process-based information and sample-based information for particular waste types. The CF is derived by dividing the concentration of a component found in the tank samples by the concentration of that component in the neutralized process waste stream (that is, the flowsheet concentrations in Table D3-1). The CF values for components of a defined waste are determined best when the tank contains only one waste type and when abundant representative analytical data are available. Multiple waste types are assumed for this tank.

The relative concentrations of components expected to precipitate essentially 100 percent to the waste solids should be approximately proportional to the respective flowsheet concentrations for those components, that is, these components should exhibit nearly the same CF values.

It was noted in the assumptions that this evaluation assumes Bi precipitated nearly 100 percent from the neutralized 1C waste, and no Bi is expected in the TBP waste. The following procedure is used to calculate the CF for Bi in tank 241-T-107. From Table D2-1, the analytical-based inventory for Bi is 11,100 kg. The solids waste volume in the tank is treated as 496 kL of 1C waste. This is a Bi concentration in the solids of 0.107*M*. The flowsheet concentration for Bi is 0.0115 *M* (Table D3-1).

The $CF_{Bi(1C)}$ is:

$$CF_{Bi(1C)} = \frac{0.107 \text{ mole Bi/L}}{0.0115 \text{ mole Bi/L}} = 9.31$$

Tank 241-T-104 has a CF_{Bi} of 10, and tank 241-BX-112 has a CF_{Bi} of 9.5. The $CF_{Bi(1C)}$ value for tank 241-T-107 appears to be reasonable. This value is used for the components assumed to fully precipitate in 1C waste. The CF value of Al is greater than that of Bi, so Al is treated as fully precipitated in the 1C waste, modifying the assumptions. The CF value of Bi is used to calculate the inventory of Al in the tank.

Once the CF values for fully precipitated components for a waste type are determined, the sample analysis can be used to establish how other components partition between solids and liquids. Concentration factors for components not expected to precipitate 100 percent in 1C waste can be ratioed to CF_{Bi} to obtain the partitioning factor (PF) for each component.

$$PF_{PO_4(1C)} = \frac{CF_{PO_4(1C)}}{CF_{Bi(1C)}} = 0.755$$

This indicates that 75.5 percent of all PO_4 added to the tank from the 1C waste stream precipitated and 24.5 percent remained in solution. The PF values of other components expected to partition in the 1C waste are as follows:

$PF_{Cr(1C)}$	0.467	$PF_{F(1C)}$	0.764
$PF_{NO_2(1C)}$	0.954	$PF_{Na(1C)}$	0.400

The CF value of Fe is used for the basis in the TBP waste. From Table D2-1, the analytical-based inventory for Fe is 27,000 kg. Assuming the iron is evenly distributed in both layers of waste, there are 6,550 kg of Fe in the TBP waste. The TBP waste solids volume in the tank is treated as 159 kL. The Fe concentration in the TBP solids is 0.738M. The flowsheet concentration for Fe is 0.03M (see Table D3-1). The $CF_{Fe(TBP)}$ is as follows:

$$CF_{Fe(TBP)} = \frac{0.738 \text{ mole Fe/L}}{0.03 \text{ mole Fe/L}} = 24.6$$

The PF values for components expected to partition in the TBP waste are as follows:

$PF_{PO_4(TBP)}$	1.012	$PF_{Na(TBP)}$	0.103
------------------	-------	----------------	-------

The PF values for tank 421-T-107 are different than those expected for 1C waste, but this tank contains a mixture of waste types. The NO_2 would appear to almost completely

precipitate and is treated as fully precipitated for this assessment. The Al was already shown to have a CF larger than that of Bi and is treated as fully precipitated. Phosphate appears to partition in the 1C waste but to precipitate fully in the TBP waste. Sodium appears to precipitate more in the 1C waste than in the TBP waste.

D3.5 CALCULATION OF ASSESSMENT-BASED INVENTORIES IN TANK 241-T-107

Assessment-based inventories are calculated in this section. These are determined by combining the assumptions with the product of the PF values calculated in the previous section, the flowsheet values in Table D3-2, the waste porosity values, and the component molecular weight. Sample calculations are shown. The components expected to partition between the solid and liquid phases are not shown.

Components assumed to precipitate (Al, Bi, Si, Zr, Fe, U, NO₂)

$$\text{Al}_{(\text{Total})}: \quad 0.0826 \text{ mole Al/L} \times 496 \text{ kL} \times 1000 \text{ L/kL} \times 27 \text{ g Al/mole} \times \text{kg/} 1000 \text{ g} \times 9.31(\text{CF}_{1\text{C}}) = 10,300 \text{ kg Al}$$

Similarly,

$$\begin{aligned} \text{Bi}_{(\text{Total})}: & \quad 11,100 \text{ kg Bi} \\ \text{Si}_{(\text{Total})}: & \quad 11,000 \text{ kg Si} \\ \text{Zr}_{(\text{Total})}: & \quad 125 \text{ kg Zr} \\ \text{NO}_{2(\text{Total})}: & \quad 12,300 \text{ kg NO}_2 \end{aligned}$$

The amounts of Fe and U are the sum of the amounts precipitated from 1C and TBP:

$$\begin{aligned} \text{Fe}_{(\text{Total})}: & \quad 14,700 \text{ kg Fe} \\ \text{U}_{(\text{Total})}: & \quad 6,740 \text{ kg U} \end{aligned}$$

Components assumed to remain dissolved in the aqueous phase (NO₃, SO₄)

$$\text{NO}_{3(1\text{C})}: \quad 1.44 \text{ mole NO}_3/\text{L} \times 496 \text{ kL} \times 1000 \text{ L/kL} \times 62 \text{ g NO}_3 / \text{mole} \times \text{kg} / 1000 \text{ g} \times 0.6948_{\text{porosity}} = 30,800 \text{ kg NO}_3$$

$$\text{NO}_{3(\text{TBP})}: \quad 7.35 \text{ mole NO}_3/\text{L} \times 159 \text{ kL} \times 1000 \text{ L/kL} \times 62 \text{ g NO}_3 / \text{mole} \times \text{kg} / 1000 \text{ g} \times 0.91417_{\text{porosity}} = 66,200 \text{ kg NO}_3$$

$$\text{NO}_{3(\text{Total})}: \quad 97,000 \text{ kg NO}_3$$

$$\text{SO}_{4(\text{Total})}: \quad 7,620 \text{ kg SO}_4$$

D3.6 COMPARISON OF SELECTED INVENTORY ESTIMATES

The estimated component inventories from this evaluation are compared with sampling and HDW model-based inventories for selected components in Table D3-3. Components that partition are not included in the table. Observations regarding these inventories are also noted.

Table D3-3. Comparison of Selected Inventory Estimates for Tank 241-T-107 Waste.

Analyte	Assessment Inventory Estimate (kg)	Sampling ¹ Inventory Estimate (kg)	HDW ² Inventory Estimate (kg)
Al	10,300	16,200	9,910
Bi	11,100	11,100	12,400
Fe	14,700	27,000	14,000
NO ₂	12,300	11,700	6,370
NO ₃	97,000	74,600	17,900
Si	11,000	6,000	1,370
SO ₄	7,620	9,860	3,320
U	6,740	22,400	105
Zr	125	112	726

Notes:

¹Table D2-1

²Agnew et al. (1996a); adjusted to a tank volume of 655 kL and density of 1.51 g/mL.

Comparison of the assessment inventory and the sampling inventory shows that 3 of the 9 constituents are very close (within 20 percent). All constituents except U are within a factor of two when the assessment inventory is compared to the sampling inventories. This is not unexpected because the evaluation was based on the sampling results and used assumptions that the 1C and TBP flowsheets each apply to only some constituents. The most notable difference between the sampling inventory and the assessment inventory is the U. The U in the sample inventory is unexpectedly high for 1C waste, but was only 30% of that value in the assessment.

The HDW inventory, adjusted to correct the waste volume and density to the sample value, was also compared to the sampling inventory. Only 1 of 9 constituents are within 20 percent, and 3 additional constituents are within a factor of two. The sample inventory is 250 times greater than the HDW inventory.

The comparisons in Table D3-2 demonstrate that adding a second waste type helps to rectify some problems caused by using only the 1C flowsheet or the HDW model to determine the inventory in tank 241-T-107. Modifying the assumptions could increase the accuracy of the assessment inventories. The HDW model assigned 159 kL of the waste as 1C, but its origin is unknown. The assessment treated this amount as TBP waste. It would be helpful to adjust the division of the total waste volume into 1C and TBP waste types. Better results may be expected if the contributions to the solids inventory of the cladding waste could be quantified.

Brief discussions about individual constituents follow.

Aluminum. The assessment and sample-based inventories are within 35 percent. The HDW inventory is slightly lower (approximately 61 percent of the sample-based inventory). This constituent was not assumed to fully precipitate in the 1C waste, although the CF indicated that it had. The higher sample value was probably influenced by Al cladding waste added to the tank late in its transfer history. This is confirmed by examining the segment level data.

Bismuth. The sampling-based inventory was used as a basis to determine the CF used for this tank. All three inventory values compare well; the HDW estimated inventory was smallest. The assessment assumed Bi to precipitate 100 percent. Bismuth was used to determine the CF for this tank. An assumption was made that Bi is evenly distributed in the 1C waste only. In reality, if the 1C waste was laid down as a sludge layer before the TBP waste was added, it would be expected that the highest concentration of Bi was at the tank bottom. Core segment data indicate that in two of three cores analyzed, this is true (that is, the Bi increases with depth, and the highest concentrations of Bi are in the lowest segments). In the third core, the highest value of Bi was in the top segment of the core; the lowest segment of the core was not retrieved.

Chromium. The assessment PF value for Cr is 0.467 indicating almost one half the Cr had precipitated. The HDW inventory is approximately 44 percent of the sample-based inventory. This constituent is expected to partition in the 1C waste.

Fluoride. The assessment PF value for F is 0.764. The HDW inventory is approximately 21 percent of the sample-based inventory. This constituent is expected to partition in the 1C waste.

Iron. The assessment and sample-based inventories are within 54 percent. The HDW inventory is slightly lower (approximately 52 percent of the sample-based inventory). This constituent is expected to fully precipitate in the 1C and the TBP wastes.

Nitrite. The assessment and sample-based inventories are within 5 percent. The HDW inventory is significantly lower (approximately 54 percent of the sample-based inventory). This constituent is expected to remain in solution.

D4.0 DEFINE THE BEST-BASIS AND ESTABLISH COMPONENT INVENTORIES

Key waste management activities include overseeing tank farm operations and identifying, monitoring, and resolving safety issues associated with these operations and with the tank wastes. Disposal activities involve designing equipment, processes, and facilities for retrieving wastes and processing them into a form that is suitable for long term storage. Information about chemical, radiological, and/or physical properties is used to perform safety analyses, engineering evaluations, and risk assessments associated with these activities.

Chemical and radiological inventory information are generally derived using three approaches: 1) component inventories are estimated using the results of sample analyses; 2) component inventories are predicted using the HDW model, process knowledge, and historical information; or 3) a tank-specific process estimate is made based on process flowsheets, reactor fuel data, essential material usage, and other operating data.

For tank 241-T-107, the sample-based inventory derived from analytical data are the best-basis inventory. Tables D4-1 and D4-2 summarize the best-basis inventory for tank 241-T-107.

Table D4-1. Best Basis Inventory Estimates for Nonradioactive Components in Tank 241-T-107. (2 sheets)

Analyte	Total Inventory (kg)	Best-Basis ¹ (S, M, or E)	Comment
Al	16,200	S	
Bi	11,100	S	
Ca	1,480	S	
Cl	541	S	
TIC as CO ₃	4,180	S	
Cr	343	S	
F	11,400	S	Based on water soluble portion only.
Fe	27,000	S	
K	231	S	
Mn	201	S	
Na	1.217E+05	S	
Ni	289	S	
NO ₂	11,700	S	
NO ₃	74,600	S	

Table D4-1. Best Basis Inventory Estimates for Nonradioactive Components in Tank 241-T-107. (2 sheets)

Analyte	Total Inventory (kg)	Best-Basis ¹ (S, M, or E)	Comment
Pb	636	S	
P as PO ₄	1.128E+05	S	
Si	6,000	S	
S as SO ₄	9,860	S	
Sr	852	S	
TOC	1,680	S	
U _{TOTAL}	22,400	S	
Zr	112	S	

Note:

¹Best-basis estimate is based on S = sampling, M = HDW model, and E = Engineering assessment.

Table D4-2. Best Basis Inventory Estimates for Radioactive Components in Tank 241-T-107.

Analyte	Total Inventory (Ci)	Best-Basis ¹ (S, M, or E)	Comment
⁹⁰ Sr	1.068E+05	S	
⁹⁰ Y	1.068E+05	S	Based on ⁹⁰ Sr
⁹⁹ Tc	50	S	
¹³⁷ Cs	12,200	S	
¹³⁷ Ba	11,500	S	Based on ¹³⁷ Cs
^{239/240} Pu	148	S	
²⁴¹ Am	13.8	S	

Note:

¹Best-basis estimate is based on S = sampling, M = HDW model, and E = Engineering assessment.

D5.0 APPENDIX D REFERENCES

- Agnew, S. F., J. Boyer, R. A. Corbin, T. B. Duran, J. R. Fitzpatrick, K. A. Jurgensen, T. P. Ortiz, and B. L. Young, 1996a, *Hanford Tank Chemical and Radionuclide Inventories: HDW Model Rev. 3*, Los Alamos National Laboratory, Los Alamos, New Mexico.
- Agnew, S. F., R. A. Corbin, T. B. Duran, K. A. Jurgensen, T. P. Ortiz, and B. L. Young, 1996b, *Waste Status and Transaction Record Summary for the Northwest Quadrant of the Hanford 200 West Area*, WHC-SD-WM-TI-669, Rev. 1, Westinghouse Hanford Company, Richland, Washington.
- Hanlon, B. M., 1996, *Waste Tank Summary Report for Month Ending October 31, 1996*, HNF-0182-103, Lockheed Martin Hanford Corporation, Richland, Washington.
- Hill, J. G., G. S. Anderson, and B. C. Simpson, 1995, *The Sort on Radioactive Waste Type Model: A Method to Sort Single-Shell Tanks into Characteristic Groups*, PNL-9814, Rev. 2, Pacific Northwest Laboratory, Richland, Washington.
- Hodgson, K. M., and M. D. LeClair, 1996, *Work Plan for Defining a Standard Inventory Estimate for Wastes Stored in Hanford Site Underground Tanks*, WHC-SD-WM-WP-311, Rev. 1, Lockheed Martin Hanford Corporation, Richland, Washington.
- Jensen, L., R. D. Cromar, and S. R. Wilmarth, 1994, *Statistical Characterization Report for Single-Shell Tank 241-T-107*, WHC-SD-WM-TI-645, Rev. 0, Westinghouse Hanford Company, Richland, Washington.
- Kupfer, M. J., 1996, *Interim Report: Best-Basis Inventories of Chemicals and Radionuclides in Hanford Site Tank Waste*, WHC-SD-WM-TI-740, Rev. D-Draft, Westinghouse Hanford Company, Richland, Washington.

This page intentionally left blank.

APPENDIX E

BIBLIOGRAPHY FOR TANK 241-T-107

This page intentionally left blank.

APPENDIX E

BIBLIOGRAPHY FOR TANK 241-T-107

Appendix E is a bibliography that supports the characterization of tank 241-T-107. This bibliography represents an in-depth literature search of all known information sources that provide sampling, analysis, surveillance, and modeling information, as well as processing occurrences associated with tank 241-T-107 and its respective waste types.

The references in this bibliography are separated into three broad categories. These categories and their subgroups are listed below.

I. NON-ANALYTICAL DATA

- Ia. Models/Waste Type Inventories/Campaign Information
- Ib. Fill History/Waste Transfer Records
- Ic. Surveillance/Tank Configuration
- Id. Sample Planning/Tank Prioritization
- Ie. Data Quality Objectives/Customers of Characterization Data

II. ANALYTICAL DATA

- IIa. Sampling of Tank Waste and Waste Types
- IIb. Sampling of 1C Waste Stream

III. COMBINED ANALYTICAL/NON-ANALYTICAL DATA

- IIIa. Inventories Using Campaign and Analytical Information
- IIIb. Compendium of Existing Physical and Chemical Documented Data Sources
- IIIc. Other Nondocumented or Electronic Sources

IV. OTHER RESOURCES

This bibliography is broken down into appropriate sections of material with an annotation at the end of each reference describing the information source. Where possible, a reference is provided for information sources. A majority of information listed below can be found in the Lockheed Martin Hanford Corporation Tank Characterization Resource Center.

I. NON-ANALYTICAL DATA

Ia. Models/Waste Type Inventories/Campaign Information

Anderson, J. D., 1990, *A History of the 200 Area Tank Farms*, WHC-MR-0132, Westinghouse Hanford Company, Richland, Washington.

- Contains single-shell tank fill history and primary campaign/waste type information up to 1981.

Babad, H., R. J. Cash, J. E. Meacham, and B. C. Simpson, 1993, *The Role of Aging in Resolving the Ferrocyanide Safety Issue*, WHC-EP-0599, Westinghouse Hanford Company, Richland, Washington.

- Contains evaluation of the effect of aging on ferrocyanide tank waste.

Borsheim, G. L., and B. C. Simpson, 1991, *An Assessment of the Inventories of the Ferrocyanide Watchlist Tanks*, WHC-SD-WM-ER-133, Rev. 0, Westinghouse Hanford Company, Richland, Washington.

- Contains brief description of ferrocyanide scavenging program and estimations of $\text{Fe}(\text{CN})_6^{4-}$, Cs-137, and Sr-90 for various ferrocyanide containing tanks.

Jungfleisch, F. M., and B. C. Simpson, 1993, *Preliminary Estimation of the Waste Inventories in Hanford Tanks Through 1980*, WHC-SD-WM-TI-057, Rev. 0A, Westinghouse Hanford Company, Richland, Washington.

- A model based on process knowledge and radioactive decay estimations using ORIGEN for different compositions of process waste streams assembled for total, solution, and solids compositions per tank. Assumptions about waste, waste types, and solubility parameters and constraints are also given.

Lilga, M. A., M. R. Lumetta, W. F. Reimath, R. A. Romine, and G. F. Schiefelbein, 1992, *Ferrocyanide Safety Project, Subtask 3.4, Aging Studies FY 1992, Annual Report*, PNL-8387, Pacific Northwest Laboratory, Richland Washington.

- Contains results of work conducted by the Pacific Northwest Laboratory in Fiscal Year 1992 on aging and solubility of ferrocyanide sludge in basic solution.

- Lilga, M. A., M. R. Lumetta, and G. F. Schiefelbein, 1993, *Ferrocyanide Safety Project, Task 3 Ferrocyanide Aging Studies FY 1993 Annual Report*, PNL-8888, Pacific Northwest Laboratory, Richland, Washington.
- Contains results of work conducted by the Pacific Northwest Laboratory in Fiscal Year 1993 on aging and solubility of ferrocyanide sludge in basic solution.
- Lilga, M. A., E. V. Aldersen, D. J. Kowalski, M. R. Lumetta, and G. F. Schiefelbein, 1994, *Ferrocyanide Safety Project, Task 3 Ferrocyanide Aging Studies FY 1994 Annual Report*, PNL-10126, Pacific Northwest Laboratory, Richland, Washington.
- Contains Fiscal Year 1994 report on ongoing ferrocyanide aging studies.
- Lilga, M. A., E. V. Aldersen, R. T. Hallen, M. O. Hogan, T. L. Hubler, G. L. Jones, D. J. Kowalski, M. R. Lumetta, G. F. Schiefelbein, and M. R. Telander, 1995, *Ferrocyanide Safety Project: Ferrocyanide Aging Studies - FY 1995 Annual Report*, PNL-10713, Pacific Northwest Laboratory, Richland, Washington.
- Contains Fiscal Year 1995 report on ongoing ferrocyanide aging studies.
- Lilga, M. A., R. T. Hallen, E. V. Aldersen, M. O. Hogan, T. L. Hubler, G. L. Jones, D. J. Kowalski, M. R. Lumetta, W. F. Riemath, R. A. Romine, G. F. Schiefelbein, and M. R. Telander, 1996, *Ferrocyanide Safety Project: Ferrocyanide Aging Studies - Final Report*, PNNL-11211, Pacific Northwest National Laboratory, Richland, Washington.
- Contains final report on ongoing ferrocyanide aging studies.
- Schneider, K. J., 1951, *Flowsheets and Flow Diagrams of Precipitation Separations Process*, HW-23043, General Electric Company, Richland, Washington.
- Contains compositions of process stream waste before transfer to 200 Area waste tanks.

Nitrate. The assessment and sample-based inventories are within 30 percent. The HDW inventory is significantly lower (approximately 24 percent of the sample-based inventory). This constituent is expected to remain in solution.

Phosphate. The PF values for PO_4 are 0.755 and 1.012 in 1C and TBP waste, respectively. In both cases, PO_4 appears to precipitate more than is expected. The HDW and sample-based inventories are within about 15 percent. This constituent is expected to partition in the waste.

Silicon. Neither the assessment nor the HDW inventories were similar to the sample-based inventory. The assessment inventory is almost twice the sampling inventory, and the HDW inventory is less than 23 percent of the sampling inventory. This constituent is expected to fully precipitate in the 1C waste.

Sodium. The assessment PF values for Na are 0.400 and 0.103 for 1C and TBP waste, respectively. This indicates the Na precipitates differently in 1C and TBP wastes. The HDW inventory is approximately 71 percent of the sample-based inventory. This constituent is expected to partition in the waste.

Sulfate. The assessment and sample-based inventories are within 30 percent. The HDW inventory is significantly lower (approximately 34 percent of the sample-based inventory). This constituent is expected to remain in solution.

Uranium. The assessment and sample-based inventories are widely separated. The assessment inventory is about 30 percent of the sample-based inventory. The HDW inventory is significantly lower (less than 1 percent of the sample-based inventory). This constituent is expected to fully precipitate in the 1C and TBP wastes.

Zirconium. The assessment and sample-based inventories are within 12 percent. The HDW inventory is significantly higher (approximately 650 percent of the sample-based inventory). This constituent is expected to fully precipitate in the 1C waste. The lack of zirconium in the sample may indicate the tank did not receive significant amounts of Zircaloy cladding waste.

D3.7 CONCLUSIONS

Based on the relationship of these three comparisons, it is obvious that the HDW model has incorrectly assigned all waste in tank 241-T-107 to 1C. The data strongly suggest that other waste types have left significant amounts of solids in the tank. With sampling data available, the sample-based inventory must be assumed to be a better estimate of the tank contents than the HDW model.

Sloat, R. J., 1954, *TBP Plant Nickel Ferrocyanide Scavenging Flowsheet*, HW-30399, General Electric Company, Richland, Washington.

- Contains compositions of process stream waste before transfer to 200 Area waste tanks.

Ib. Fill History/Waste Transfer Records

Agnew, S. F., R. A. Corbin, T. B. Duran, K. A. Jurgensen, T. P. Ortiz, and B. L. Young, 1996, *Waste Status and Transaction Record Summary for the Northwest Quadrant of the Hanford 200 East Area*, WHC-SD-WM-TI-669, Rev. 1, Westinghouse Hanford Company, Richland, Washington.

- Contains spreadsheets showing all available data on tank additions and transfers.

Anderson, J. D., 1990, *A History of the 200 Area Tank Farms*, WHC-MR-0132, Westinghouse Hanford Company, Richland, Washington.

- Contains single-shell tank fill history and primary campaign and waste type information up to 1981.

Ic. Surveillance/Tank Configuration

Alstad, A. T., 1993, *Riser Configuration Document for Single-Shell Waste Tanks*, WHC-SD-RE-TI-053, Rev. 9, Westinghouse Hanford Company, Richland, Washington.

- Shows tank riser locations in relation to tank aerial view as well as a description of riser and its contents.

Hanlon, B. M., 1996, *Tank Farm Surveillance and Waste Status Summary Report for Month Ending October 31, 1996*, HNF-EP-0182-103, Lockheed Martin Hanford Corporation, Richland, Washington.

- Most recent release of a series of summaries including fill volumes, Watch List tanks, occurrences, integrity information, equipment readings, equipment status, tank location, and other miscellaneous tank information. The series includes monthly summaries from December 1947 to the present, however Hanlon has only authored the monthly summaries from November 1989 to the present.

Hodgson, K. M., and M. D. LeClair, 1996, *Work Plan for Defining a Standard Inventory Estimate for Wastes Stored in Hanford Site Underground Tanks*, WHC-SD-WM-WP-311, Rev. 1, Lockheed Martin Hanford Corporation, Richland, Washington.

- Contains strategy to define an inventory estimate for tank wastes.

Leach, C. E. and S. M. Stahl, 1997, *Hanford Site Tank Farm Facilities Interim Safety Basis Volume I and II*, WHC-SD-WM-ISB-001, Rev. 0M, Duke Engineering and Services Hanford, Richland, Washington.

- Provides a ready reference to the tank farms safety envelope.

Lipnicki, J., 1996, *Waste Tank Risers Available for Sampling*, WHC-SD-WM-TI-710, Rev. 3, Westinghouse Hanford Company, Richland, Washington.

- Gives an assessment of riser locations for each tank; however, not all tanks are included or completed. Also includes an estimate of which risers are available for sampling.

Tran, T. T., 1993, *Thermocouple Status Single-Shell & Double-Shell Waste Tanks*, WHC-SD-WM-TI-553, Rev. 0, Westinghouse Hanford Company, Richland, Washington.

- Compiles information on thermocouple trees installed in Hanford Site underground waste tanks.

Welty, R. K., 1988, *Waste Storage Tank Status and Leak Detection Criteria, Volumes I and II*, WHC-SD-TI-553, Rev. 0, Westinghouse Hanford Company, Richland, Washington.

- Describes the nature, scope, and frequency of surveillance employed for waste storage tanks, states action criteria for response to data deviation, and reviews tank data between June 15, 1973 and June 15, 1988.

WHC, 1987, *Quarterly Trend Analysis of Surveillance Data*, (internal memorandum 65950-87-587 to R. J. Baumhardt, June 29), Westinghouse Hanford Company, Richland, Washington.

- Third quarter trend analysis of waste tank surveillance data to identify trends or anomalies.

Id. Sample Planning/Tank Prioritization

- Bell, K. E., 1993, *Tank Waste Remediation System Tank Waste Characterization Plan*, WHC-SD-WM-PLN-047, Rev. 1, Westinghouse Hanford Company, Richland, Washington.
- Details a plan providing a partially integrated approach to the characterization of the Hanford Site tank wastes. The scope of this plan is defined by the characterizing activities for safely storing, maintaining, treating, and disposing of tank waste onsite or packaging it for offsite.
- Brown, T. M., S. J. Eberlein, J. W. Hunt, and T. J. Kunthara, 1996, *Tank Waste Characterization Basis*, WHC-SD-WM-TA-164, Rev. 2, Westinghouse Hanford Company, Richland, Washington.
- Summarizes the technical basis for characterizing tank waste and assigns a priority number to each tank.
- Carpenter B. C., 1995, *Tank 241-T-107 Tank Characterization Plan*, WHC-SD-WM-TP-295, Rev. 0A Westinghouse Hanford Company, Richland, Washington.
- Contains detailed sampling and analysis procedure information for vapor sampling of tank 241-T-107.
- Ecology, EPA, and DOE, 1996, *Hanford Federal Facility Agreement and Consent Order*, as amended, Washington State Department of Ecology, U.S. Environmental Protection Agency, and U.S. Department of Energy, Olympia, Washington.
- Contains Tri-party agreement for the Hanford site.
- Hill, J. G., 1991, *Modified Test Plan for Core Sample Analysis of FeCN SSTs C-112, C-109, and T-107*, (internal memorandum 9158449 to J. H. Kessner, November 11), Westinghouse Hanford Company, Richland, Washington.
- Contains modified and revised analytical plan stated in Winters et al. (1991). Sampling of tank 241-T-107 was initiated according to this memo, but sampling was completed and analyses were performed per Bell (1993).

Homi, C. S., 1995, *Tank 241-T-107 Vapor Sampling and Analysis Plan*, WHC-SD-WM-TP-335, Rev. 0G, Westinghouse Hanford Company, Richland, Washington.

- Contains a discussion of tank vapor sampling and the sampling and analysis that will be needed for tank 241-T-107.

Winkelman, W. D., J. W. Hunt, and L. J. Fergestrom, 1996, *FY 1997 Tank Waste Analysis Plan*, WHC-SD-WM-PLN-120, Rev. 1, Westinghouse Hanford Company, Richland, Washington.

- Contains Tri-Party Agreement requirement-driven TWRS Characterization Program information and a list of tanks addressed in Fiscal Year 1997.

Keller, K. K., 1994, *Quality Assurance Project Plan for Tank Vapor Characterization*, WHC-SD-WM-QAPP-013, Rev. 2, Westinghouse Hanford Company, Richland, Washington

- Contains specific quality assurance requirements.

Rich, H. S., 1993, *Sampling and Analysis of SST and DST Waste Tanks in Support of TWRS Fiscal Year 1993, Statement of Work*, WHC-SOW-93-0002, Rev. 1, Westinghouse Hanford Company, Richland, Washington.

- Document formally transmits a request to the laboratories to analyze core samples in accordance with Bell (1993) and includes various letters of instruction.

Smith, H. E., 1992, *Technical Project Plan - Response to WHC-SOW-91-0006 for the 222-S Analytical Laboratory*, WHC-SD-CP-TP-070, Rev. 0, Westinghouse Hanford Company, Richland, Washington.

- Documentation for laboratory work in support of Winters et al. (1991).

Winters, W. I., 1992, *Technical Project Plan for 222-S Laboratory in Support of Tank Waste Remediation System Tank Waste Characterization Plan (WHC-SD-WM-PLN-047, Rev. 0) and Statement of Work (WHC-SOW-93-0002)*, WHC-SD-WM-TPP-047, Rev. 0, Westinghouse Hanford Company, Richland, Washington.

- Provides the 222-S Laboratory response to the statement of work. It describes how the laboratory plans to analyze the samples and lists exceptions.

Winters, W. I., J. G. Hill, B. C. Simpson, J. W. Buck, P. J. Chamberlain, and V. L. Hunter, 1991, *Waste Characterization Plan for the Hanford Site Single-Shell Tanks*, WHC-EP-0210, Rev. 3, Westinghouse Hanford Company, Richland, Washington.

- This revision added *Appendix I Test Plan for Sampling and Analysis of Ten Single-Shell Tanks* and revised Appendix D (the quality assurance project plan).

Ie. Data Quality Objectives and Customers of Characterization Data

Cowan, S. P., 1996, *Approval to Remove 14 Ferrocyanide Tanks from the Watch List*, (internal memorandum to J. Kinzer, U.S. Department of Energy, Richland Operations Office, August 21), U.S. Department of Energy, Washington D.C.

- Contains justifications for removing tanks from Ferrocyanide Watch List.

Dukelow, G. T., J. W. Hunt, H. Babad, and J. E. Meacham, 1995, *Tank Safety Screening Data Quality Objective*, WHC-SD-WM-SP-004, Rev. 2, Westinghouse Hanford Company, Richland, Washington.

- Contains objectives to sample all tanks for safety concerns (ferrocyanide, organic, flammable gas, and criticality) as well as decision thresholds for energetics, criticality, and flammability.

Kinzer, J., 1996, *Authorization to Remove the Remaining 14 Ferrocyanide Tanks 241-BY-103, 241-BY-104, 241-BY-105, 241-BY-106, 241-BY-107, 241-BY-108, 241-BY-110, 241-BY-110, 241-BY-112, 241-T-107, 241-TX-118, 241-TY-101, 241-TY-103 and 241-TY-104 from the Watch List*, (letter 9602303 to A. L. Trego, Westinghouse Hanford Company, September 4), U.S. Department of Energy, Richland Operations Office, Richland, Washington.

- Contains authorization for removing tanks from Ferrocyanide Watch List.

Meacham, J. E., R. J. Cash, B. A. Pulsipher, and G. Chen, 1995, *Data Requirements for the Ferrocyanide Safety Issue Developed Through the Data Quality Objective Process*, WHC-SD-WM-DQO-007, Rev. 2, Westinghouse Hanford Company, Richland, Washington.

- Contains ferrocyanide program data needs, list of tanks to be evaluated, decision thresholds, and decision logic flow diagram.

Osborne, J. W., J. L. Huckaby, T. P. Rudolph, E. R. Hewitt, D. D. Mahlum, J. Y. Young, and C. M. Anderson, 1994, *Data Quality Objectives for Generic In-Tank Health and Safety Issue Resolution*, WHC-SD-WM-DQO-002, Rev. 0, Westinghouse Hanford Company, Richland, Washington.

- Contains program data needs, list of tanks to be evaluated, decision thresholds, and decision logic flow diagram.

Postma, A. K., G. S. Barney, G. L. Borsheim, R. J., Cash, M. D. Crippen, D. R. Dickinson, J. M. Grigsby, D. W. Jeppson, C. S. Simmons, and B. C. Simpson, 1994, *Ferrocyanide Safety Program: Safety Criteria for Ferrocyanide Watchlist Tanks*, WHC-EP-0691, Westinghouse Hanford Company, Richland, Washington.

- Document which was developed before the safety screening and ferrocyanide DQOs specifying safety criteria.

II. ANALYTICAL DATA - SAMPLING OF TANK WASTE AND WASTE TYPES

IIa. Sampling of Tank 241-T-107

Bratzel, D. R., 1985, *Tank 107-T Waste Mixing Study*, (internal letter 65453-85-043 to L. A. Gale, March 5), Rockwell Hanford Company, Richland, Washington.

- Provides sampling and mixing waste study results from the March 5 sampling of supernate.
- Bratzel, D. R., 1989, *Interim Results of T-101 and T-107 Analysis and T-101/Neutralized Plutonium Finishing Plant Acid Waste*, (internal memorandum 12712-PCL89-144 to A. J. DiLiberto, August 1), Westinghouse Hanford Company, Richland, Washington.
- Provides sampling and mixing waste study results from the sampling of several tanks.
- Caprio, G. S., 1995, *Vapor and Gas Sampling of Single-Shell Tank 241-T-107 Using the Vapor Sampling System*, WHC-SD-WM-RPT-130, Rev. 0, Westinghouse Hanford Company, Richland, Washington.
- Provides results from the January 1995 sampling of the tank headspace (a complete revision was later written by Huckaby and Bratzel).
- Colton, N. G., 1996, *Status Report: Pretreatment Chemistry Evaluation - Wash and Leach Factors for the Single-Shell Tank Waste Inventory*, PNNL-11290, Pacific Northwest National Laboratory, Richland, Washington.
- Contains sludge wash data for all single-shell tanks evaluated since 1986 including tank 241-T-107.
- Godfrey, W. L., 1965, *242-T Evaporator Feed*, (internal memorandum to S. J. Beard, September 24), General Electric, Richland, Washington.
- Contains results of analysis of prospective feed for 242-T evaporator including a sample from tank 241-T-107.
- Huckaby and Bratzel, D. R., 1995, *Tank 241-T-107 Headspace Gas and Vapor Characterization Results for Samples Collected in January 1995*, WHC-SD-WM-ER-447, Rev. 0, Westinghouse Hanford Company, Richland, Washington.
- Contains specific headspace gas and vapor characterization results for all vapor sampling events to date. In addition, changes have been made to the original vapor reports to qualify the data based on quality assurance issues associated with the performing laboratories.
- Klem, M. J., 1990, *Total Organic Carbon Concentration of Single-Shell Waste*, (internal memorandum 82316-90-032 to R. E. Raymond, April 27), Westinghouse Hanford Company, Richland, Washington.

- Summarizes and gives references for the TOC values for 47 single-shell tanks based on available laboratory analysis of solid and/or liquid waste samples.
- Jensen, L., R. D. Cromar, and S. R. Wilmarth, 1994, *Statistical Characterization Report for Single-Shell Tank 241-T-107*, WHC-SD-WM-TI-645, Rev. 0, Westinghouse Hanford Company, Richland, Washington.
- Contains the results of the statistical analysis of data from three core samples from the 1992 and 1993 core sampling.
- Lumetta, G. L., M. J. Wagner, S. V. Hoops, and R. T. Steele, 1996, *Washing and Caustic Leaching of Hanford Tank C-106 Sludge*, PNNL-11381, Pacific Northwest National Laboratory, Richland, Washington.
- Contains data on the samples taken for privatization in 1996. In support of providing privatization vendors with washed tank 241-C-106 sludge for high-level waste vitrification studies, a pretreatment screening study was performed on about 15 g of material.
- Pingel, L. A., 1992, *Waste Tank T-107 Vapor Sampling Results*, (internal memorandum 12240-SAS93-003 to G. L. Dukelow, November 2), Westinghouse Hanford Company, Richland, Washington.
- Contains results of the October 22, 1992 vapor sampling.
- Pool, K. H., R. B. Lucke, B. D McVeety, G. S Klinger, T. W. Clauss, M. W. Ligothke, K. B. Olsen, O.P. Bredt, J. S. Fruchter, and S. C. Goheen, 1995, *Vapor Space Characterization of Waste Tank 241-T-107: Results from Samples Collected on 1/18/95*, PNL-10595, Pacific Northwest National Laboratory, Richland, Washington.
- Contains specific headspace gas and vapor characterization results for the January 18, 1995 vapor sampling event.
- Sasaki, L. M., and B. D. Valenzuela, 1994, *Ferrocyanide Safety Program: Data Interpretation Report for Tank 241-T-107 Core Samples*, WHC-EP-0796, Rev. 0, Westinghouse Hanford Company, Richland, Washington.
- Contains sample analyses from 1992 and 1993 tank 241-T-107 core sampling event and evaluates the waste in accordance with the ferrocyanide safety program.

Svancara, G. B. and K. N. Pool, 1993, *WHC 222-S and PNL-325 Single-Shell Tank Waste Characterization, 241-T-107 Cores 50, 51, and 52 - Data Package and Validation Summaries*, WHC-SD-WM-DP-042, Rev. 1A, Westinghouse Hanford Company, Richland, Washington.

- Contains sample analyses and data validation from 1992 and 1993 tank 241-T-107 core sampling event.

Valenzuela, B. D., and L. Jensen, 1994, *Tank Characterization Report for Single-Shell Tank 241-T-107*, WHC-SD-WM-ER-382, Rev. 0A, Westinghouse Hanford Company, Richland, Washington.

- Expands the evaluation of data for the sample analyses from 1992 and 1993 tank 241-T-107 core sampling event.

Wheeler, R. E., 1975, *Analysis of Tank Farm Samples, Sample: T-8248, Tank 107-T, Received: September 25, 1975*, (internal memorandum to R. L. Waiser, November 26), Atlantic Richfield Hanford Company, Richland, Washington.

- Contains sample analysis results for a liquid sample from tank 241-T-107.

Iib. Sampling of 1C Waste Stream

This section provides sampling data for other tanks containing 1C waste. The tank 241-T-104 data are particularly significant because some basic assumptions for the best-basis inventory for tank 241-T-107 are based on this data.

Conner, J. M., 1996, *Final Report for Tank 241-BX-112, Auger Samples 95-AUG-047 and 95-AUG-048*, WHC-SD-WM-DP-157, Rev. 0, Westinghouse Hanford Company, Richland, Washington.

- Provides analytical results from the 1995 auger sampling event.

Duchsherer, M. J., 1993, *Single-Shell Tank Waste Characterization 241-T-104, Core 45 and 46 Narrative*, WHC-SD-WM-DP-032, Rev. 0, Westinghouse Hanford Company, Richland, Washington.

- Provides analytical results from the 1992 core sampling event.

Schreiber, R. D., 1995, *45-Day Safety Screening Results and Final Report for Tank 241-BX-110, Auger Samples 95-AUG-045 and 95-AUG-046*, WHC-SD-WM-DP-155, Rev. 0, Westinghouse Hanford Company, Richland, Washington.

- Provides analytical results from the 1995 auger sampling event.

III. COMBINED ANALYTICAL/NON-ANALYTICAL DATA

IIIa. Inventories from Campaign and Analytical Information

Agnew, S. F., J. Boyer, R. A. Corbin, T. B. Duran, J. R. Fitzpatrick, K. A. Jurgensen, T. P. Ortiz, and B. L. Young, 1996, *Hanford Tank Chemical and Radionuclide Inventories: HDW Model Rev. 3*, LA-UR-96-858, Rev. 0, Los Alamos National Laboratory, Los Alamos, New Mexico.

- Contains waste type summaries, primary chemical compound/analyte and radionuclide estimates for sludge, supernatant, and solids, as well as supernatant mixing model, tank layer model, and individual tank inventory estimates.

Agnew, S. F., 1995, *Letter Report: Strategy for Analytical Data Comparisons to HDW Model*, (Letter CST-4:95-sfa272 to Susan Eberlein, September 28), Los Alamos National Laboratory, Los Alamos, New Mexico.

- Contains proposed tank groups based on tank layer model and a statistical method for comparing analytical information to HDW predictions.

Allen, G. K., 1976, *Estimated Inventory of Chemicals Added to Underground Waste Tanks, 1944 - 1975*, ARH-CD-601B, Atlantic Richfield Hanford Company Operations, Richland, Washington.

- Contains major components for waste types, and some assumptions.

Allen, G. K., 1975, *Hanford Liquid Waste Inventory As Of Sept. 30, 1974*, ARH-CD-229, Atlantic Richfield Hanford Company Operations, Richland, Washington.

- Contains major components for waste types, and some assumptions.

Geier, R. G., 1976, *Estimated Hanford Liquid Wastes Chemical Inventory as of June 30, 1976*, ARH-CD-768, Atlantic Richfield Hanford Company, Richland, Washington.

- Estimates contents of a number of single-shell tanks in support of the waste solidification program.

Grigsby, J. M., 1992, *Ferrocyanide Waste Tank Hazard Assessment - Interim Report*, WHC-SD-WM-RPT-032, Rev. 0, Westinghouse Hanford Company, Richland, Washington.

- Contains inventory estimates from physical and campaign data for a few constituents in ferrocyanide containing tanks and a few laboratory analyses.

Jensen, L., R. D. Cromar, and S. R. Wilmarth, 1994, *Statistical Characterization Report for Single-Shell Tank 241-T-107*, WHC-SD-WM-TI-645, Rev. 0, Westinghouse Hanford Company, Richland, Washington.

- Documents the statistical analyses performed on core sample data from the 1993 sampling of tank 241-T-107

Schmittroth, F. A., 1995, *Inventories for Low-Level Tank Waste*, WHC-SD-WM-RPT-164, Rev. 0, Westinghouse Hanford Company, Richland, Washington.

- Contains a global inventory based on process knowledge and radioactive decay estimations using ORIGEN2. Pu and U waste contributions are taken at one percent of the amount used in processes. Also compares information on Tc-99 from ORIGEN2 and analytical data.

IIIb. Compendium of data from other sources physical and chemical

Agnew, S. F., and J. G. Watkin, 1994, *Estimation of Limiting Solubilities for Ionic Species in Hanford Waste Tank Supernates*, LA-UR-94-3590, Los Alamos National Laboratory, Los Alamos, New Mexico.

- Document gives solubility ranges used for key chemical and radionuclide components based on supernatant sample analyses.

Brevick, C. H., L. A. Gaddis, and E. D. Johnson, 1995, *Historical Tank Content Estimate for the Northwest Quadrant of the Hanford 200 Areas*, WHC-SD-WM-ER-351, Rev. 0, Westinghouse Hanford Company, Richland, Washington.

- Contains summary information from the supporting document for Tank Farms T, TX, and TY as well as in-tank photographic collages and the solid (including the interstitial liquid) composite inventory estimates.

Brevick, C. H., L. A. Gaddis, and W. W. Pickett, 1995, *Supporting Document for the Northwest Quadrant Historical Tank Content Estimate for T-Tank Farm*, WHC-SD-WM-ER-320, Rev. 0, Westinghouse Hanford Company, Richland, Washington.

- Contains summary tank farm information and tank write-ups on historical data and solid inventory estimates as well as appendixes for the data. The appendixes contain the following information: temperature graphs, surface level graphs, cascade/drywell charts, riser configuration drawings and tables, in-tank photos, and tank layer model bar charts and spreadsheets.

Brevick, C. H., L. A. Gaddis, and E. D. Johnson, *Tank Waste Source Term Inventory Validation, Volume I & II*, WHC-SD-WM-ER-400, Rev. 0, Westinghouse Hanford Company, Richland, Washington.

- Contains sampling information in spreadsheet or graphical form for 23 chemicals and 11 radionuclides for all tanks.

De Lorenzo, D. S., J. H. Rutherford, D. J. Smith, D. B. Hiller, K. W. Johnson, and B. C. Simpson, 1994, *Tank Characterization Reference Guide*, WHC-SD-WM-TI-648, Rev. 0A, Westinghouse Hanford Company, Richland, Washington.

- Summarizes issues surrounding characterization of nuclear wastes stored in Hanford Site waste tanks.

Hanlon, B. M., 1996, *Tank Farm Surveillance and Waste Status Summary Report for Month Ending October 31, 1996*, HNF-EP-0182-103, Lockheed Martin Hanford Corporation, Richland, Washington.

- Contains a summary of fill volumes, Watch List tanks, occurrences, integrity information, equipment readings, equipment status, tank location, and other miscellaneous tank information. Grouped here are

all the monthly summaries from December 1947 to the present; however, Hanlon has only authored the monthly summaries from November 1989 to the present.

Hartley, S. A., G. Chen, T. A. Ferryman, A. M. Liebetrau, K. M. Remund, and S. A. Allen, 1996, *A Comparison of Historical Tank Contents Estimates (HTCE) Model, Rev. 3, and Sample-Based Estimates*, PNNL-11429, Pacific Northwest National Laboratory, Richland, Washington.

- Contains statistical comparisons of historical model inventories to sample-based inventories.

Husa, E. I., R. E. Raymond, R. K. Welty, S. M. Griffith, B. M. Hanlon, R. R. Rios, and N. J. Vermeulen, 1993, *Hanford Site Waste Storage Tank Information Notebook*, WHC-EP-0625, Rev. 0, Westinghouse Hanford Company, Richland, Washington.

- Contains in-tank photographs and summaries on the tank description, leak detection system, and tank status.

Husa, E. I., 1995, *Hanford Waste Tank Preliminary Dryness Evaluation*, WHC-SD-WM-TI-703, Rev. 0, Westinghouse Hanford Company, Richland, Washington.

- Assesses relative dryness among tanks.

Jungfleisch, F. M., 1980, *Hanford High-Level Defense Waste Characterization - A Status Report*, RHO-CD-1019, Rockwell Hanford Operations, Richland, Washington.

- Provides status information to plan outlined by G. W. Grimes, October 1977, containing a summary of sampling, characterization, and analysis data for the tanks sampled.

Remund, K. M., and B. C. Simpson, 1996, *Hanford Waste Tank Grouping Study*, PNL-11433, Pacific Northwest National Laboratory, Richland, Washington.

- Contains multivariable statistical study categorizing tanks into groups based on analytical data.

Shelton, L. W., 1996, *Chemical and Radionuclide Inventory for Single- and Double-Shell Tanks*, (internal memorandum 74A20-96-30 to D. J. Washenfelder, February 28), Westinghouse Hanford Company, Richland, Washington.

- Contains a tank inventory estimate based on analytical information.

Shelton, L. W., 1995, *Chemical and Radionuclide Inventory for Single- and Double-Shell Tanks*, (internal memorandum 75520-95-007 to R. M. Orme, August 8), Westinghouse Hanford Company, Richland, Washington.

- Contains a tank inventory estimate based on analytical information.

Shelton, L. W., 1995, *Radionuclide Inventories for Single- and Double-Shell Tanks*, (internal memorandum 71320-95-002 to F. M. Cooney, February 14), Westinghouse Hanford Company, Richland, Washington.

- Contains a tank inventory estimate based on analytical information.

IIIc. Other - Nondocumented or Electronic Sources

Pacific Northwest National Laboratory, 1997, TWINS: Tank Waste Information Network System. In: SYBASE version 4. Available: Hanford Local Area Network (HLAN), Lockheed Martin Services, Richland, Washington; or TCP/IP access, Pacific Northwest National Laboratory, Richland, Washington.

- Provides access to Surveillance Analysis Computer System, Tank Monitor and Control System, Tank Characterization Database, and Kaiser electronic data.

Pacific Northwest National Laboratory, 1997, TCD: Tank Characterization Database. In: SYBASE version 4.0. Available: Tank Waste Information Network System (TWINS), Pacific Northwest National Laboratory, Richland, Washington

- Contains qualified raw sampling data taken in the past few years from 222-S Laboratory. A small amount of information from the 325 Laboratory data is included at this time.

IV. OTHER RESOURCES

Fluor Daniel Northwest, 1997, Fluor Daniel Northwest Tank Characterization Library. In hard copy. Available: Fluor Daniel Northwest, 200E, Trailer MO-971 Room 26, Sheryl Consort: custodian, Fluor Daniel Northwest, Richland, Washington.

- A resource of 200 Area tank, process campaign, reactor, and other historical records, unclassified and declassified.

WHC, 1995, 222-S Laboratory RIDS: Records Inventory and Disposition Schedule. In: Hardcopy. Available: In 222-S Laboratory RIDS index, Westinghouse Hanford Company archives, Westinghouse Hanford Company, Richland, Washington.

- A RIDS report of the information archived for 1992 and 1993 from the 222-S Laboratory, last printed May 17, 1995. Laboratory notebooks may have been archived that contain pertinent information.

LMHC, 1997, L.S.I.S.: Large Scale Information System, ERS DB - Engineering Release Station Database. In: Database. Available: Hanford Local Area Network (HLAN), Lockheed Martin Hanford Corporation, Richland, Washington.

- Database with any released document information. Most expedient to search by title and keyword for tank in question.

Lockheed Martin Services, 1997, RMIS: Record Management Information System, Records Database. In: Database. Available: HLAN, Lockheed Martin Services, Richland, Washington.

- Records is a database of all released documents since November 1995, which will be back loaded with previous year's data. It can be queried to find documents for any subject either in the keyword or description field.

Lockheed Martin Services, 1997, RMIS: Record Management Information System, Tank Farm Information Center Database. In: Database. Available: HLAN, Lockheed Martin Services, Richland, Washington.

- TFIC is a database of tank related reports, memorandums, and letters that have been optically scanned. The database can be queried to find indexed information for a tank [in the tank or description field] or information referenced to any subject either in the keyword or description field.

LMHC, 1997, TCRC: Tank Characterization Resource Center. In: hard copy. Available: 2750E Room A-243, Ann Young: custodian, Lockheed Martin Hanford Corporation, Richland, Washington.

- A resource of TWRS characterization data including the following: hard copy file folders of sampling data for each tank, an index of multiple tank documents folders, physical/chemical data compendiums, and studies or reports on 200 Area Tanks or Tank Waste generated by various contractors.

WHC, 1996, 209-E Waste Tanks Document Index. In: Hard copy. Available: Fluor Daniel Northwest Library, Fluor Daniel Northwest, Richland, Washington.

- An index of general and tank-specific information for 200 Area tanks.

This page intentionally left blank.

DISTRIBUTION SHEET

To Distribution	From Data Assessment and Interpretation	Page 1 of 3
		Date 03/10/97
Project Title/Work Order Tank Characterization Report for Single-Shell Tank 241-T-107, HNF-SD-WM-ER-382, Rev. 1		EDT No. N/A
		ECN No. ECN-635447

Name	MSIN	Text With All Attach.	Text Only	Attach./Appendix Only	EDT/ECN Only
------	------	-----------------------	-----------	-----------------------	--------------

OFFSITE

Sandia National Laboratory
P. O. Box 5800
MS-0744, Dept. 6404
Albuquerque, NM 87815

D. Powers X

Nuclear Consulting Services Inc.
P. O. Box 29151
Columbus, OH 43229-01051

J. L. Kovach X

Chemical Reaction Sub-TAP
P.O. Box 271
Lindsborg, KS 67456

B. C. Hudson X

Tank Characterization Panel
Senior Technical Consultant
Contech
7309 Indian School Road
Albuquerque, NM 87110

J. Arvisu X

SAIC
20300 Century Boulevard, Suite 200-B
Germantown, MD 20874

H. Sutter X

Los Alamos Laboratory
CST-14 MS-J586
P. O. Box 1663
Los Alamos, NM 87545

S. F. Agnew X

DISTRIBUTION SHEET

To Distribution	From Data Assessment and Interpretation	Page 2 of 3
		Date 03/10/97

Project Title/Work Order Tank Characterization Report for Single-Shell Tank 241-T-107, HNF-SD-WM-ER-382, Rev. 1	EDT No. N/A
	ECN No. ECN-635447

Name	MSIN	Text With All Attach.	Text Only	Attach./Appendix Only	EDT/ECN Only
------	------	-----------------------	-----------	-----------------------	--------------

Los Alamos Technical Associates

T. T. Tran B1-44 X

Tank Advisory Panel

102 Windham Road
Oak Ridge, TN 37830

D. O. Campbell X

ONSITE

Department of Energy - Richland Operations

J. F. Thompson S7-54 X
W. S. Liou S7-54 X
J. A. Poppiti S7-54 X
N. W. Willis S7-54 X

DE&S Hanford, Inc.

R. J. Cash S7-14 X
W. L. Cowley R2-54 X
G. L. Dunford A2-34 X
G. D. Johnson S7-14 X
J. E. Meacham S7-14 X

Fluor Daniel Northwest

J. L. Stroup S3-09 X

Lockheed Martin Hanford, Corp.

K. M. Hodgson H0-34 X
T. J. Kelley S7-21 X
L. M. Sasaki R2-12 X
B. C. Simpson R2-12 X
L. R. Webb R2-12 X
ERC (Environmental Resource Center) R1-51 X
Tank Characterization Resource Center R2-12 5

Lockheed Martin Services, Inc.

B. G. Lauzon R1-08 X
Central Files A3-88 X
EDMC H6-08 X

DISTRIBUTION SHEET

To Distribution	From Data Assessment and Interpretation	Page 3 of 3
		Date 03/10/97
Project Title/Work Order Tank Characterization Report for Single-Shell Tank 241-T-107, HNF-SD-WM-ER-382, Rev. 1		EDT No. N/A
		ECN No. ECN-635447

Name	MSIN	Text With All Attach.	Text Only	Attach./ Appendix Only	EDT/ECN Only
<u>Numatec Hanford Corporation</u>					
J. S. Garfield	H5-49	X			
J. S. Hertzell	H5-61	X			
D. L. Lamberd	H5-61	X			
<u>Pacific Northwest National Laboratory</u>					
A. F. Noonan	K9-91	X			
<u>Rust Federal Services of Hanford, Inc.</u>					
C. T. Narquis	T6-16	X			
<u>SGN Eurisys Services Corp.</u>					
D. B. Engelman	L6-37	X			